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Measurements of decay heat in spent nuclear fuel at the Swedish interim storage facility, Clab

Svensk Kärnbränslehantering AB

December 2006

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Measurements of decay heat in spent nuclear fuel at the Swedish interim storage facility, Clab

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Measurements of decay heat in spent nuclear fuel at the Swedish central interim storage facility, Clab

Calorimetric and gamma radiation measurements on spent nuclear fuel have been performed at the Swedish central interim storage facility, Clab, on 50 BWR fuel assemblies and 34 PWR fuel assemblies during 2004.

The purpose with the measurements is to develop a quick and simple method to determine the decay heat in spent nuclear fuel prior to encapsulation and final disposal.

The equipment and the measurements are described in this report, which consists of five parts:

Part 1. Clab – Measurements of decay heat in spent nuclear fuel assemblies

In this part the calorimetric measurements are summarized. Measurement results are presented and a comparison between measured and calculated values is accounted for.

Part 2. Clab – Gamma scanning of spent nuclear fuel assemblies: determination of residual thermal power using new data acquisition and analysis software

The gamma scanning is presented in this part. The equipment, measurements and results are presented.

Part 3. Clab – Calibration curve for calorimetric measurements of fuel assemblies

Development of the calibration curve for the calorimeter is presented in this part.

Part 4. Clab – Uncertainty analysis of the calorimeter (system 251) using the temperature increase method

An uncertainty analysis of the calorimeter and the measured decay heat values is performed in this part.

Part 5. Clab – Data for fuel assemblies used in calorimetric and nuclear measurements

In this part the fuel data for the spent nuclear fuel is presented as received from the power plants.

Clab- Measurements of decay heat in spent nuclear fuel assemblies

Fredrik Sturek, Lennart Agrenius

Contents

1	Introduction	9
2	Calorimetric method	11
2.1	General	11
2.2	Description of the calorimeter	11
2.3	Measurement principles	12
2.4	Calibration	14
2.5	Heat loss due to gamma radiation	16
3	Calorimetric measurements on spent nuclear fuel	17
3.1	General	17
3.2	Measurement program	17
3.3	Measurements	19
4	Calculations	25
4.1	Codes	25
4.2	Fuel data	25
4.3	Results	25
5	Conclusions	29
	References	31
	Appendix 1 Power and temperature values in the calibration curve	33
	Appendix 2 F-factors for gamma escape-calculations	35
	Appendix 3 Correction factors to compensate for thermal capacity difference between electric heater and BWR fuel assemblies	37
	Appendix 4 Correction factors to compensate for thermal capacity difference between electric heater and PWR fuel assemblies	39
	Appendix 5 Raw data from the measurements BWR	41
	Appendix 6 Raw data from the measurements PWR	43
	Appendix 7 Raw data from gamma measurements	45

1 Introduction

Svensk Kärnbränslehantering AB (SKB) is conducting measurements of decay heat and gamma radiation on spent fuel assemblies in the Swedish central interim storage facility for spent nuclear fuel (Clab).

The objectives of these measurements are:

Primary:

- Accurate and simple determination of decay power in final disposal canisters for spent nuclear fuel by use of correlation with gamma measurements on fuel assemblies.
- Verification of burnup and cooling time of spent nuclear fuel prior to encapsulation.
- More accurate prediction of total decay power in Clab.

Secondary:

- More accurate decay power prediction prior to fuel transport to Clab.
- Calibration of decay heat calculations by e.g. Origen and Decay (Swedish).
- Provide a basis for BU-credit (if needed) in final disposal canister.

This report describes the measurements and calculations on 50 BWR – and 34 PWR – assemblies performed so far (end of 2004).

Measurements will be repeated in certain intervals in the future on the same selected fuel assemblies. The population may later be increased by assemblies of different designs.

2 Calorimetric method

2.1 General

A calorimeter was designed with the purpose to measure the decay heat power in spent nuclear fuel assemblies of PWR- and BWR-types.

Decay heat power from 50 W to 1,000 W should be possible to measure.

The design goal was that the uncertainty of the decay heat power measurement should be less than $\pm 2\%$.

The equipment is designed to be located under water in one of the pools in the Clab reception area. Fuel assemblies are placed in and removed from the equipment with the normal fuel handling system in Clab.

2.2 Description of the calorimeter

The calorimeter is 5 m long and is composed of two concentric pipes. The inner pipe, which has an inner diameter of 0.33 m forms the test chamber. The test chamber is 4.5 m long. The space between the inner and outer cylinders is filled with polyuretan foam, which acts as thermal insulation between the test chamber and the pool water.

A lid is placed at the top of the calorimeter. The purpose of the lid is to form a leak tight barrier between the test chamber and the pool water. The lid is opened, closed, locked and unlocked with help of compressed air.

The calorimeter is normally placed in the service pool in the reception area at the Clab facility.

The fuel assembly is placed in the calorimeter with the normal fuel handling equipment in Clab. In the test chamber there is a fixed insert for PWR fuel and a removable insert for BWR-fuel. The inserts maintain the fuel assembly in centred vertical position.

A centrifugal pump circulates water in the calorimeter in order to maintain a homogenous temperature in the test chamber. The nominal capacity of the pump is 60 l/min. A flow through the test chamber can be established with help of a measurement flow pump with a controllable flow in the range of 0–20 l/min. The measurement flow is measured by a flow-measuring unit, which can be calibrated by means of a calibration tank in which the mass of water is measured by a scale.

In the calorimeter the temperatures are measured with 16 PT-100 sensors. Eight sensors are placed in the water inside the test chamber. Two sensors are placed on the inside and outside surfaces of the calorimeter, and two sensors are placed in the water outside the calorimeter to measure the pool water temperature. Temperatures of the measurement flow are measured at the incoming and outgoing flow of the calorimeter.

Outside the calorimeter five gamma radiation monitors are placed on a movable arm to measure the radial distribution of the gamma radiation that escapes from the calorimeter. The arm is movable axially in 10 fixed positions and can be turned 90 degrees around the calorimeter.

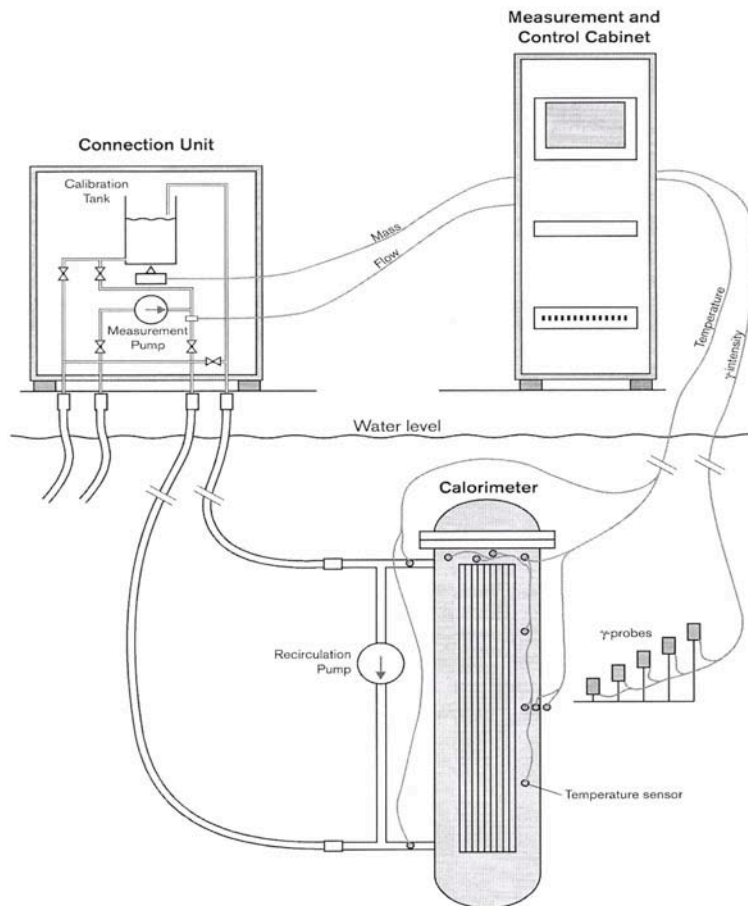


Figure 2-1. Calorimeter.

Calibration of the calorimeter is performed with an electric heater designed in the same shape as a BWR fuel assembly.

Valves and pumps are controlled by a PC-based system. All data are logged by the PC-system.

2.3 Measurement principles

The basic measurement principle is to measure the temperature increase of the water in the calorimeter caused by the decay heat power from a fuel assembly placed in the calorimeter.

The decay heat power is determined by comparing the temperature increase to a calibration curve. Before each measurement campaign a calibration curve is established using an electric heater. The temperature increase in the calorimeter is measured for several power levels. Based on these measurements a calibration curve is established. Calibration curves could be established for the three different measurement methods.

The three different methods to measure the temperature increase are:

1. Temperature increase method.
2. Recirculation method.
3. Equilibrium method.

Temperature increase method

A fuel assembly is placed in the calorimeter. The lid is closed. In order to assure that the calorimeter is completely full with water the measurement flow pump is operated until water is flowing out into the calibration tank. The pump is stopped and the water is freely communicating with the calorimeter thus creating an overpressure in the calorimeter. The circulation pump is started and the temperature increase is monitored. After an increase of around 4°C the measurement is stopped. The measured increase rate of the temperature is compared to the corresponding calibration measurements and the decay power is determined. The gamma radiation field is measured and the power escaped with the gamma radiation is determined. The sum of the measured decay power and power from the escaped gamma gives the total decay heat power.

Recirculation method

A fuel assembly is placed in the calorimeter. The lid is closed. In order to assure that the calorimeter is completely full with water the measurement flow pump is operated until water is flowing out into the calibration tank. Then the recirculation flow is redirected back to the pool. In this case there is a flow through the calorimeter, which will be heated up. When equilibrium is reached the measurement is stopped. Based of the flow, the temperature difference between incoming and outgoing water and the gamma escape the decay heat power could be evaluated.

Equilibrium method

A fuel assembly is placed in the calorimeter. The lid is closed. In order to assure that the calorimeter is completely full with water the measurement flow pump is operated until water is flowing out into the calibration tank. The pump is stopped and the water is freely communicating with the calorimeter thus creating an overpressure in the calorimeter. Then circulation pump is started and the temperature increase is monitored. The temperature is increasing until equilibrium is reached.

The temperature difference between starting temperature and equilibrium temperature is compared to the calibration curve. This will give the decay power in the calorimeter. Correction for gamma escape will give the total decay heat power.

Comparison between the methods

Tests have shown that the equilibrium method is very time consuming. It takes at least one week to establish equilibrium in the calorimeter (one measurement point). More than 20 measurement points are required to get a good calibration curve and around 20 assemblies are planned to be measured each year. This method would give too long measurement times and will not be used.

With the temperature increase and the circulation methods one measurement point takes around 4–5 hours for calibration. Fuel assemblies takes longer because time is needed for fuel handling. Realistically one fuel assembly could be measured in one day.

The circulation method shows also to be time consuming and requires more attention from the operator than the other methods.

The conclusion of these experiences is that the temperature increase method will be used.

2.4 Calibration

To be able to translate the temperature increase into decay power a calibration curve has been established, see also part 3. An immersion heater was used to heat the volume of water in the calorimeter. The heater was designed to be similar to a BWR-fuel assembly using structural parts from a fuel assembly. A heating cable was attached in the structure to simulate the fuel assembly geometry.

The heating cable was powered from a stabilised power supply and the power input was measured by a wattmeter. The measurement data is automatically recorded.

The heater is placed in the calorimeter and the heater power is adjusted to a preset value. Deionised water is circulated through the calorimeter in order to cool it to around 1.5°C below the pool water temperature.

The circulation of ionised water is stopped and the temperature in the calorimeter will start to increase. Logging of the temperatures, time and other parameters is started. The logging is stopped after a certain temperature increase

The temperature difference between the calorimeter and the pool water is calculated by subtracting the average value of the temperature sensors inside from average value of the temperature sensors outside the calorimeter. Presently two sensors inside and two sensors outside the calorimeter are used.

A second degree polynomial is correlated to the temperature difference and time. The slope of the curve at the zero temperature difference is given by the coefficient of the first degree term in the equation.

The procedure is repeated at several power levels of the electric heater. The result is a table with slope values and corresponding power values. From this table a calibration curve is generated by correlating a strait line to the slope and power values. The correlation gives the equation of this strait line.

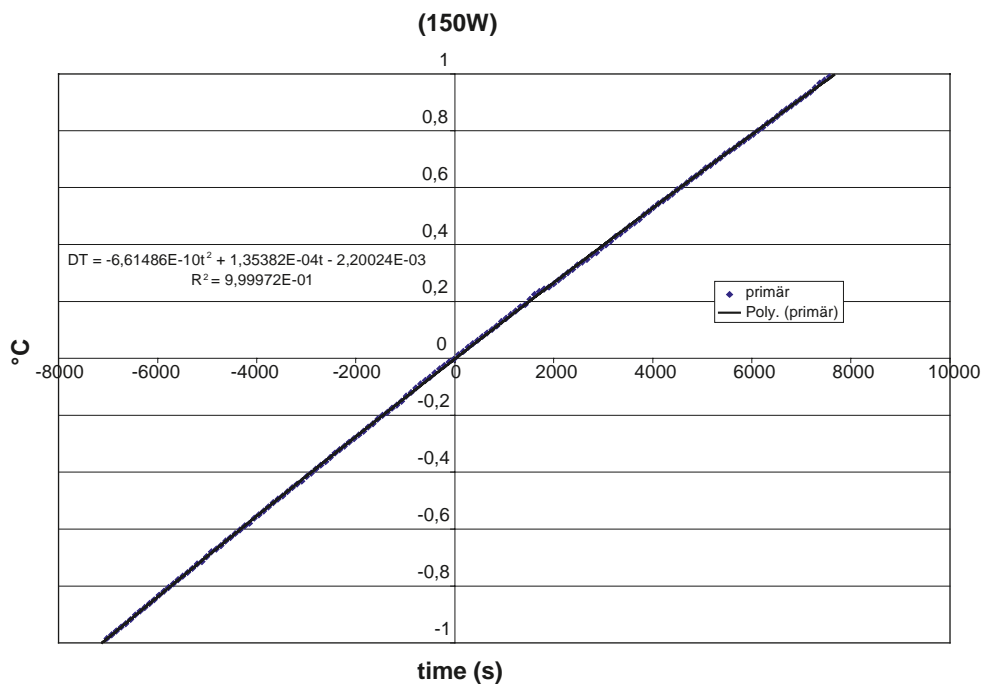


Figure 2-2. Diagram – sample temperature increase during calibration with 150 W.

In total 51 measurements have been carried out at a power input range of 49 to 735 watts. These measurements have been used to generate calibration curves for the BWR- and the PWR- cases.

By fitting a straight line using the least squares method to all the heat rate values and powers, an equation for the calibration curve is obtained. The general equation is

$$P_{\text{cal}} = a + b \, dt/dT \text{ where}$$

P_{cal} = power in the heater at calibration

a and b = constants from the curve fitting

dt/dT = measured rate of temperature increase

For BWR the power is expected to be between 60 and 350 W and the fit between these values gives the following equation for BWR:

$$P_{\text{cal}} = 1.73 \times 10^6 \times dt/dT - 90.0$$

For PWR a calibration curve is generated for powers between 260 and 950 W. The resulting equation is:

$$P_{\text{cal}} = 1.72 \times 10^6 \times dt/dT - 86.4$$

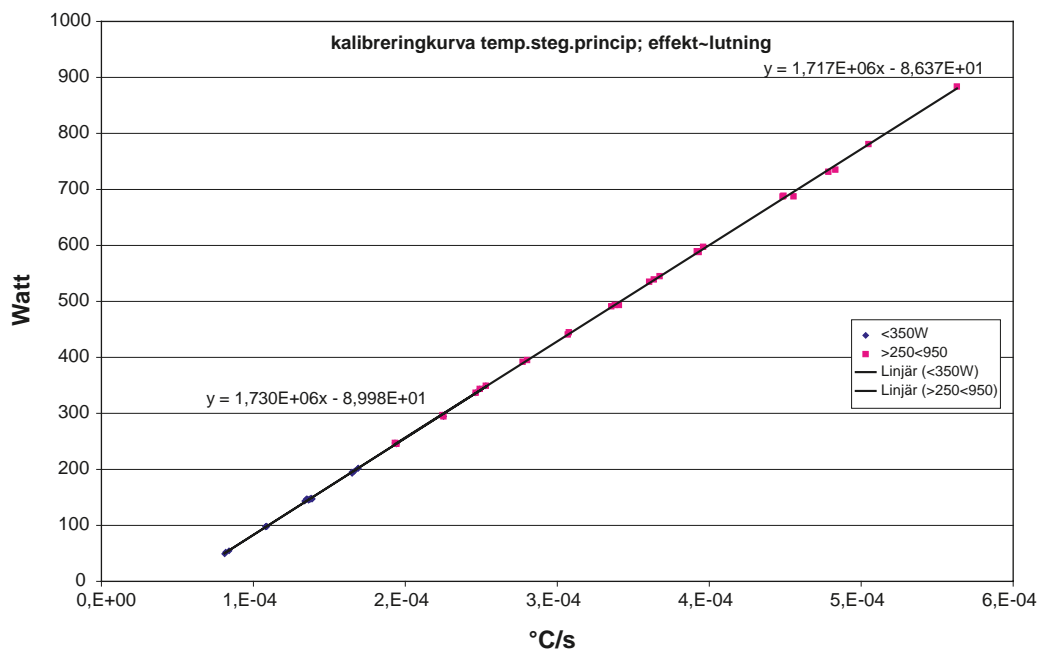


Figure 2-3. Diagram – Calibration curve.

2.5 Heat loss due to gamma radiation

A fraction of the decay heat from the fuel assembly is lost from the calorimetric measurement by the gamma radiation that escapes from the calorimeter.

The measured decay heat values have to be corrected accordingly. Gamma intensity measurements are used to assess this effect.

To accomplish that gamma radiation from the fuel are measured with five gamma probes at the outside of the calorimeter. The gamma probes are at the distance from the center of the vessel as presented in Table 2-1.

A relation between gamma escape power P_γ , measured gamma dose rate (Gy/h) d_i and the gamma probes placing r_i is shown in reference 1. The relation is as follows:

$$P_\gamma = a \frac{\sum_{i=1}^5 d_i e^{-\lambda r_i}}{\sum_{i=1}^5 (e^{-\lambda r_i})^2} \left\{ e^{-\lambda} \left(-\frac{1}{\lambda^2} - \frac{1}{\lambda} \right) + e^{-0.228\lambda} \left(\frac{1}{\lambda^2} + \frac{0.228}{\lambda} \right) \right\} F \quad \text{Equation 2-1}$$

$a = 8.14$ W for BWR and 8.41 W for PWR.

$\lambda = 8.4$ m⁻¹ attenuation coefficient.

r_i = The gamma probes distance from the center according.

F = Profile factor, unique for each fuel assembly. In Appendix 2 the F-factors for the fuel assemblies are found.

For each fuel measurement the gamma dose rate are measured with the probes d_1 – d_5 . The gamma dose rate is used in the relation above and the gamma escape power is possible to calculate

Table 2-1. Distance between the center of the vessel and the gamma probe.

Probe d_i	Radius (m)
1	0.26
2	0.38
3	0.50
4	0.63
5	0.77

3 Calorimetric measurements on spent nuclear fuel

3.1 General

When measuring a fuel assembly the following procedure is followed. An assembly is placed in the calorimeter (the heater is removed before). The calorimeter is cooled down with deionised water. After the circulation is stopped, logging of the temperatures, time, gamma radiation rate and other parameters is started. Logging is stopped after a certain temperature increase.

The temperature difference between the calorimeter of the pool is calculated by subtracting the average value of the temperature sensors inside with average value of the temperature sensors outside the calorimeter. Presently two sensors inside and two sensors outside the calorimeter are used.

A second degree curve is correlated to the temperature difference and time. The slope of the curve at zero temperature difference is given by the coefficient of the first degree term in the equation.

A correction of the slope is done to account for the fact that the fuel assembly has different volume and contains different materials than the electric heater.

The resulting slope is used in the calibration curve equation, which is solved for the power. With the measured gamma dose rates the power that escapes the calorimeter with the gamma radiation is calculated using the equation given above.

The sum of the power in the calorimeter and the power corresponding to the gamma field will give the decay heat power for the measured fuel assembly.

3.2 Measurement program

50 BWR and 34 PWR assemblies were selected for measurement from the Clab inventory. The selected assemblies met the following criteria:

- Fuel types PWR: 15×15, 17×17 and BWR: 8×8, Svea64, Svea100 were included.
- Assemblies were gamma scanned in this measurement campaign and fuel has long cooling times.
- The assemblies possessed a large spread in burnup.
- The assemblies possessed a large spread in initial enrichment.
- The assemblies possessed long cooling times.
- The assemblies did not contain inserts as boron glass rods, neutron sources or other parts.
- The assemblies were not mechanically damaged or had leaking rods

Main data for the selected fuel is presented in Table 3-1 for BWR fuel and Table 3-2 for PWR fuel.

Table 3-1. BWR fuel.

Assembly no	Box	Fuel type	Reactor	Enrichment (%U235)	Burnup (MWd/tU)	Initial weight (g U)	Shutdown date
10288	Yes	8x8	B1	2.95	35,180	179,159	1988-09-16
2014	Yes	8x8	B1	2.33	19,648	179,878	1979-04-13
2018	Yes	8x8	B1	2.33	20,605	179,852	1980-09-03
2048	Yes	8x8	B1	2.33	22,559	179,741	1979-04-13
2074	Yes	8x8	B1	2.33	22,923	179,657	1980-09-03
2118	Yes	8x8	B1	2.50	20,654	179,607	1978-05-09
9329	Yes	8x8	B1	2.92	41,094	178,771	1988-09-16
14076	Yes	8x8	B2	3.15	40,010	179,571	1992-07-09
3838	Yes	8x8	F1	2.09	25,669	177,903	1992-07-10
KU0100	Yes	8x8-2	F1	2.98	34,193	174,920	1990-09-01
KU0269	Yes	9x9-5	F1	2.94	35,113	177,020	1990-09-02
KU0278	Yes	9x9-5	F1	2.94	35,323	177,133	1991-06-08
KU0282	Yes	9x9-5	F1	2.94	37,896	177,097	1991-06-06
11494	Yes	Svea 64	F2	2.92	32,431	181,088	1988-07-25
11495	Yes	Svea 64	F2	2.91	32,431	181,070	1988-07-27
13775	Yes	Svea 64	F2	2.85	32,837	181,340	1991-07-18
5535	Yes	8x8	F2	2.10	19,944	177,689	1988-07-15
13847	Yes	Svea 100	F3	2.77	31275	180,669	1990-07-22
13848	Yes	Svea 100	F3	2.77	31275	180,667	1990-07-22
12684	Yes	Svea 64	O2	2.90	46,648	182,320	1991-08-05
1377	Yes	8x8	O2	2.20	14,546	183,575	1977-05-20
1389	Yes	8x8	O2	2.20	19,481	183,650	1981-07-23
1546	Yes	8x8	O2	2.20	24,470	183,968	1983-08-25
1696	Yes	8x8	O2	2.20	20,870	184,253	1983-08-25
1704	Yes	8x8	O2	2.20	19,437	184,022	1982-07-30
2995	Yes	8x8	O2	2.70	29,978	179,382	1981-07-25
3054	Yes	8x8	O2	2.89	34,893	160,262	1983-08-25
3058	Yes	8x8	O2	2.89	31,987	160,372	1983-08-25
3064	Yes	8x8	O2	2.89	30,391	160,318	1982-07-30
6350	Yes	8x8	O2	2.88	27,675	179,003	1985-06-14
12078	Yes	8x8	O3	2.58	25,160	177,404	1988-07-08
13628	Yes	Svea 100	O3	2.71	35619	180,774	1991-06-28
13630	Yes	Svea 100	O3	2.71	40363	180,775	1991-06-28
0582	No	8x8	R1	2.26	21,270	177,394	1980-09-15
0596	No	8x8	R1	2.26	22,256	177,199	1980-09-15
0710	No	8x8	R1	2.26	22,614	177,308	1982-09-27
0900	No	8x8	R1	2.26	23,152	177,362	1982-09-27
1136	No	8x8	R1	2.26	22,230	171,581	1983-08-23
1177	Yes	8x8	R1	2.65	36,242	180,587	1985-09-12
1186	Yes	8x8	R1	2.65	30,498	180,515	1985-09-12
5829	Yes	8x8	R1	2.71	44,861	156,410	1987-08-27
6423	No	8x8	R1	2.90	35,109	177,701	1988-09-05
6432	No	8x8	R1	2.89	36,861	177,520	1988-09-05
6454	No	8x8	R1	2.90	37,236	177,683	1986-07-18
6478	No	8x8	R1	2.90	35,183	126,675	1986-07-18
8327	No	8x8	R1	2.90	37,851	177,544	1991-08-15
8331	No	8x8	R1	2.91	35,903	177,690	1989-09-21
8332	No	8x8	R1	2.90	34,977	177,519	1988-08-13
8338	No	8x8	R1	2.91	34,830	177,596	1988-08-13
8341	No	8x8	R1	2.89	34,099	174,113	1988-08-13

Table 3-2. PWR fuel.

Assembly no	Fuel type	Reactor	Enrichment (%U235)	Burnup (MWd/tU)	Initial weight (g U)	Shutdown date
C01	15x15	R2	3.10	36,688	455,789	1981-05-17
C12	15x15	R2	3.10	36,385	453,736	1981-05-16
C20	15x15	R2	3.10	35,720	454,758	1985-04-04
C42	15x15	R2	3.10	35,639	453,923	1988-05-12
D27	15x15	R2	3.25	39,676	432,589	1983-06-13
D38	15x15	R2	3.25	39,403	434,214	1982-05-27
E38	15x15	R2	3.20	33,973	433,593	1982-05-29
E40	15x15	R2	3.20	34,339	434,244	1982-05-28
F14	15x15	R2	3.20	34,009	436,382	1983-06-12
F21	15x15	R2	3.20	36,273	435,939	1984-04-23
F25	15x15	R2	3.20	35,352	437,281	1983-06-12
F32	15x15	R2	3.20	50,962	436,993	1988-05-22
G11	15x15	R2	3.19	35,463	436,180	1985-05-02
G23	15x15	R2	3.21	35,633	436,125	1985-05-02
I09	15x15	R2	3.20	40,188	437,353	1988-05-12
I20	15x15	R2	3.20	34,313	423,896	1986-06-30
I24	15x15	R2	3.20	34,294	429,597	1986-06-30
I25	15x15	R2	3.20	36,859	433,062	1987-04-25
OC9	17x17	R3	3.10	38,442	457,639	1986-06-18
OE2	17x17	R3	3.10	41,628	463,598	1988-07-07
OE6	17x17	R3	3.10	35,993	461,769	1988-07-07
1C2	17x17	R3	3.10	33,318	459,050	1986-06-17
1C5	17x17	R3	3.10	38,484	457,992	1986-06-17
1E5	17x17	R3	3.10	34,638	463,898	1988-07-07
2A5	17x17	R3	2.10	20,107	462,026	1984-05-25
2C2	17x17	R3	3.10	36,577	459,490	1986-06-17
3C1	17x17	R3	3.10	36,572	458,433	1986-06-17
3C4	17x17	R3	3.10	38,447	456,170	1986-06-18
3C5	17x17	R3	3.10	38,373	458,873	1986-06-17
3C9	17x17	R3	3.10	36,560	459,138	1986-06-17
4C4	17x17	R3	3.10	33,333	459,050	1986-06-17
4C7	17x17	R3	3.10	38,370	458,256	1986-06-17
5A3	17x17	R3	2.10	19,699	461,477	1984-05-24
5F2	17x17	R3	3.40	47,308	451,743	1991-06-20

3.3 Measurements

Totally 109 measurements have been carried out. Eight assemblies were measured twice, two assemblies three times, one four times, one five times, and one seven times. The procedure for decay heat determination was as follow.

The fuel assembly to be measured was lowered into the vessel and the lid was mounted over the opening of the calorimeter.

Deionized water is connected to the vessel to cool it down to 1.5°C below the surrounding water in the basin.

Data was logged down until the temperature was 1.5°C above the surrounding water.

The difference in temperature between the inside of the vessel and the surrounding water was plotted against time, see Figure 3-1 which shows, as an example, the measuring of fuel assembly number 8341 date 040312. The time zero is the time when the temperature difference between the vessel and the surrounding basin is zero. An equation of second degree is fitted in the best way to the line. The slope in the time zero point is defined (1.488×10^{-4} in Figure 3-1).

The fuel assembly and the electric heater, used to produce the calibration curve, have not the same volume and material content. This means that the amounts of energy to increase the temperature one degree differ between the calibration and the fuel assembly measurement. To compensate for the difference in thermal capacity, a correction factor is used, which is presented in appendices 3 and 4. In the tables, the volume, amount of zirconium, steel/inconel, and uranium dioxide can also be seen. This information is used when the correction factors are calculated. The calculations of the correction factors for fuel assembly 6432, 9329 and 5A3 are presented in detail in part 3.

Appendix 3 gives the correction factor for fuel assembly 8341 to 0.972 implying a corrected slope value of 1.446×10^{-4} .

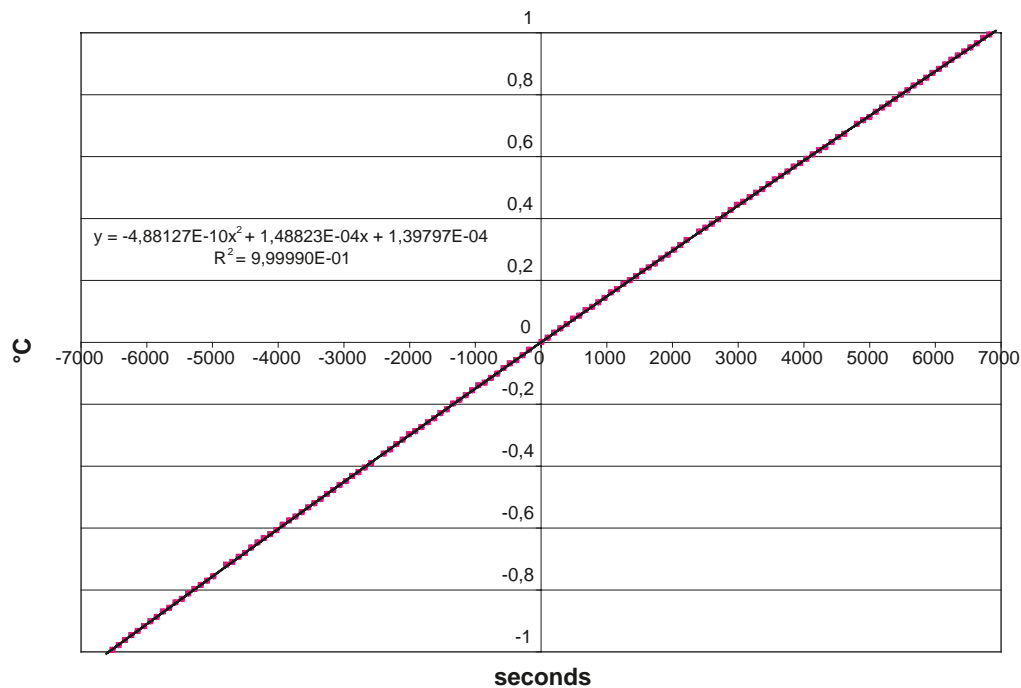


Figure 3-1. Diagram – Temperature versus time of fuel assembly 8341.

The corrected slope value was used to determine the decay power for the fuel assembly, by using the equations presented above. The equations are:

$P = 1.73 \times 10^6 \times (\text{corrected slope}) - 90.0$ for fuel assembly with a power below 350 W for BWR-fuel.

For fuel assembly with an power between 250 Watt and 950 Watt (PWR-fuel) the equation is: $P = 1.72 \times 10^6 \times (\text{corrected slope}) - 86.4$.

In appendices 5 and 6 the slope value, corrected slope value and the decay power for all measured fuel assembly are presented.

Measurements on BWR fuel

In total 66 measurements have been carried out. Six assemblies were measured twice, two assemblies three times and one seven times.

The results of the measurement and the evaluated decay powers including correction for the heat loss due to gamma radiation are presented in Table 3-3 for the BWR-assemblies:

Table 3-3. Results from measurements BWR-assemblies.

Assembly no	Fuel type	Measured decay heat (W)					Average	Gamma escape (W)	Total decay heat (W)
		1	2	3	4	5			
582	8x8	89.3					89.3	2.3	91.7
596	8x8	85.3	88.2				86.8	3.0	89.7
710	8x8	93.3	89.9	89.3			90.9	3.0	93.9
900	8x8	93.7					93.7	2.8	96.5
1136	8x8	92.6	93.2				92.9	2.6	95.5
1177	8x8	173.4					173.4	4.5	177.9
1186	8x8	137.5					137.5	3.3	140.8
1377	8x8	54.8					54.8	1.4	56.2
1389	8x8	81.7					81.7	2.2	83.9
1546	8x8	105.4					105.4	2.7	108.1
1696	8x8	90.0					90.0	2.3	92.4
1704	8x8	81.8					81.8	2.1	84.0
2014	8x8	80.8					80.8	1.9	82.7
2018	8x8	82.3					82.3	2.0	84.3
2048	8x8	92.4					92.4	2.2	94.6
2074	8x8	95.5					95.5	2.3	97.8
2118	8x8	95.9					95.9	2.4	98.3
2995	8x8	127.3					127.3	3.2	130.5
3054	8x8	137.4					137.4	3.7	141.0
3058	8x8	123.3					123.3	3.5	126.7
3064	8x8	118.6					118.6	3.2	121.7
3838	8x8	123.4	122.6				123.0	3.4	126.4
5535	8x8	82.4					82.4	2.2	84.6

Assembly no	Fuel type	Measured decay heat (W)					Average	Gamma escape (W)	Total decay heat (W)
		1	2	3	4	5			
5829	8x8	205.2					205.2	5.5	210.7
6350	8x8	123.7	126.2				124.9	3.2	128.2
6423	8x8	169.1					169.1	5.2	174.2
6432x	8x8	179.9	184.0				182.0	5.5	187.5
6432xx	8x8	178.9	177.1	179.4	175.8	176.4	177.5	5.5	183.1
6454	8x8	181.3					181.3	5.0	186.3
6478	8x8	117.8					117.8	3.7	121.5
8327	8x8	191.0					191.0	6.0	196.9
8331	8x8	181.3					181.3	5.7	187.0
8332	8x8	163.0					163.0	5.1	168.1
8338	8x8	164.9					164.9	4.6	169.5
8341	8x8	160.1	158.1				159.1	4.8	163.9
9329*	8x8	217.4	218.9				218.2	5.4	223.6
9329**	8x8	213.2					213.2	5.4	218.7
10288	8x8	181.2					181.2	4.6	185.8
11494	Svea 64	161.7					161.7	4.4	166.0
11495	Svea 64	163.3					163.3	4.3	167.6
12078	8x8	116.9					116.9	3.3	120.2
12684	Svea 64	274.9					274.9	7.8	282.7
13628	Svea 100	188.2					188.2	5.8	194.0
13630	Svea 100	228.6					228.6	7.1	235.7
13775	Svea 64	173.1					173.1	5.3	178.4
13847	Svea 100	165.4	164.7				165.0	4.9	169.9
13848	Svea 100	165.6					165.6	5.1	170.7
14076	8x8	233.7					233.7	6.6	240.3
KU0100	8x8-2	180.2					180.2	5.1	185.3
KU0269	9x9-5	187.1					187.1	5.6	192.7
KU0278	9x9-5	189.9					189.9	5.5	195.4
KU0282	9x9-5	212.5					212.5	5.9	218.5

*) measured 2003, **) measured 2004

Five measurements were carried out for assembly 6432 between 13/2 – 23/2 2004. The average power is 177.5 W with the standard deviation of 1.6 W (1%).

Measurements on PWR fuel

34 PWR assemblies were measured. In total 43 measurements were carried out, two assemblies were measured twice, one four times and one five times.

The following results were obtained for the PWR-assemblies.

Table 3-4. Results from PWR-measurements.

Assembly no	Fuel type	Measured decay heat (W)					Average	Gamma escape (W)	Total decay heat (W)
		1	2	3	4	5			
0C9	17x17	478.6					478.6	12.6	491.2
0E2	17x17	572.4					572.4	15.5	587.9
0E6	17x17	475.1					475.1	12.6	487.8
1C2	17x17	407.3					407.3	10.4	417.7
1C5	17x17	486.5					486.5	12.7	499.2
1E5	17x17	456.9					456.9	11.9	468.8
2A5	17x17	227.9					227.9	5.8	233.8
2C2	17x17	454.8					454.8	11.7	466.5
3C1	17x17	458.3					458.3	11.9	470.2
3C4	17x17	484.9					484.9	12.4	497.3
3C5	17x17	488.9					488.9	12.5	501.4
3C9	17x17	456.5					456.5	11.9	468.4
4C4	17x17	411.4					411.4	10.6	422.0
4C7	17x17	485.9					485.9	12.8	498.7
5A3*	17x17	231.9	230.8	237.6			233.4	5.8	239.3
5A3**	17x17	225.1	224.4				224.8	5.8	230.6
5F2	17x17	695.5					695.5	18.5	714.1
C01	15x15	406.3					406.3	9.4	415.8
C12	15x15	401.6					401.6	8.7	410.3
C20	15x15	406.1	416.4	419.2	425.9		416.9	9.7	426.6
C42	15x15	431.7	437.8				434.7	10.6	445.3
D27	15x15	444.6					444.6	11.4	456.1
D38	15x15	431.6					431.6	10.8	442.3
E38	15x15	366.9	364.9				365.9	9.4	375.3
E40	15x15	372.2					372.2	9.0	381.3
F14	15x15	372.5					372.5	9.3	381.8
F21	15x15	410.2					410.2	10.7	420.9
F25	15x15	386.7					386.7	10.0	396.7
F32	15x15	675.4					675.4	16.6	692.0
G11	15x15	405.7					405.7	10.6	416.4
G23	15x15	409.6					409.6	11.1	420.6
I09	15x15	494.5					494.5	13.4	507.9
I20	15x15	393.8					393.8	9.7	403.5
I24	15x15	398.2					398.2	11.8	410.1
I25	15x15	434.0					434.0	11.8	445.8

*) measured 2003, **) measured 2004

Four measurement were carried out for assembly C20 during the period 14/4– 23/4 2004. The average power is 416.9 W with the standard deviation of 8.2 W (2%).

4 Calculations

4.1 Codes

The decay heats for the current assemblies have been calculated using the SAS2 sequence in the SCALE code system. SAS2 is sequence designed to calculate comprehensive nuclide concentrations, radiation source terms, and decay heat generation for spent nuclear fuel. The fuel burnup and decay analysis, and calculation of the neutron and gamma ray sources is performed by the ORIGEN-S code. A key feature of SAS2 is the ability to generate problem-dependent cross section for the burnup analysis based on user-specified assembly design characteristics, fuel type, reactor operating conditions, and irradiation and decay history. Cross sections for the reactor fuel assembly are developed using the one-dimensional (1-D) neutron transport analysis code XSDRNPM using a two-part procedure with two separate lattice-cell models. This procedure provides a pseudo 2-D analysis capability using 1-D transport methods that is suitable to many commercial LWR and research reactor fuel designs. The cross sections for the burnup analysis derived from the transport analysis are applied in an ORIGEN-S point-depletion computation to produce the burnup-dependent fuel compositions that are used in the next spectrum calculation. The cross sections are generated using the neutron flux spectrum for the assembly model for the specified burnup-dependent fuel compositions, reference 2.

4.2 Fuel data

Fuel type, enrichment, initial uranium weight, burnup and shutdown date for all fuel assemblies can be seen in Tables 3-1 and 3-2. Detailed fuel data and irradiation history of each assembly are presented in part 5.

Concerning the power history the number of cycles is correctly modelled, but the detailed power history within each cycle is not represented. Each assembly has been assigned a mean value for the entire cycle. Power changes during the cycles and during coast-down operation have consequently not been modelled, other than in the mean value.

4.3 Results

BWR

The following Table 4-1 compares the BWR-measurements and the calculations.

Table 4-1. Comparison between measured and calculated values for BWR.

Assembly no	Measured decay heat (W)	Calculated decay heat (W)
582	91.7	89.7
596	89.7	94.3
710	93.9	97.1
900	96.5	100.1
113	695.5	97.4
1177	177.9	177.4
1186	140.8	143.1
1377	56.2	59.5
1389	83.9	77.4
1546	108.1	112.1
1696	92.4	84.3
1704	84.0	85.0
2014	82.7	83.3
2018	84.3	86.8
2048	94.6	94.9
2074	97.8	98.5
2118	98.3	88.9
2995	130.5	132.1
3054	141.0	144.9
3058	126.7	130.6
3064	121.7	121.7
3838	126.4	129.8
5535	84.6	92.0
5829	210.7	204.3
6350	128.2	131.3
6423	174.2	176.9
6432*	187.5	187.8
6432**	183.1	185.1
6454	186.3	182.2
6478	121.5	119.9
8327	196.9	201.2
8331	187.0	184.6
8332	168.1	173.6
8338	169.5	172.9
8341	163.9	168.9
9329*	223.6	223.2
9329**	218.7	220.7
10288	185.8	184.6
11494	166.0	165.7
11495	167.6	166.0
12078	120.2	133.7
12684	282.7	276.1
13628	194.0	193.9
13630	235.7	228.5
13775	178.4	178.2
13847	169.9	166.6
13848	170.7	166.4
14076	240.3	249.6
KU0100	185.3	173.6
KU0269	192.7	179.8
KU0278	195.4	182.3
KU0282	218.5	197.9

*) measured 2003, **) measured 2004

Figure 4-1 shows a comparison between the measured value and the calculated value by Scale. The calculations show excellent agreement with the measurements on the average. The average ratio between calculated and measured data is 1.001.

PWR

The measured and calculated values are shown in Table 4-2.

Figure 4-2 shows a comparison between the measured value and the calculated value by Scale. The calculations show good agreement with the measurements on the average. The calculations seem overestimate the power with 2% on average.

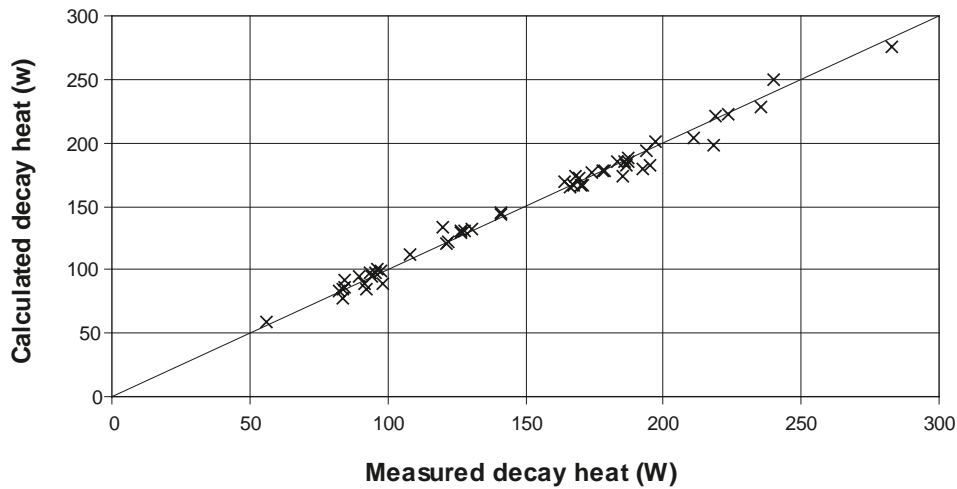


Figure 4-1. BWR-comparison between calculated and measured decay heat.

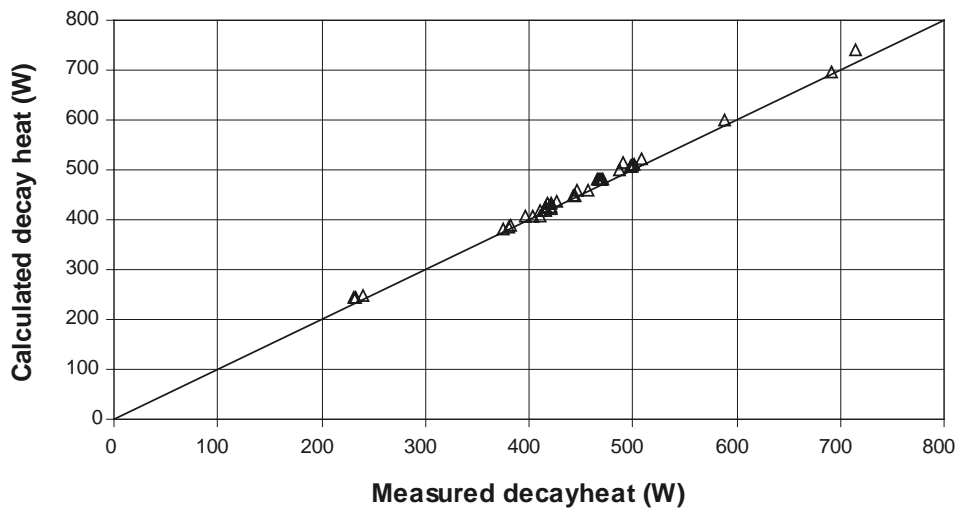


Figure 4-2. PWR-comparison between calculated and measured values.

Table 4-2. Comparison between measured and calculated values PWR.

Assembly no	Measured decayheat (W)	Calculated decayheat (W)
0C9	491.2	514.4
0E2	587.9	599.4
0E6	487.8	501.1
1C2	417.7	431.9
1C5	499.2	510.1
1E5	468.8	482.1
2A5	233.8	243.4
2C2	466.5	482.3
3C1	470.2	480.9
3C4	497.3	509.2
3C5	501.4	509.3
3C9	468.4	481.2
4C4	422.0	433.8
4C7	498.7	508.2
5A3*	239.3	248.2
5A3**	230.6	244.2
5F2	714.1	742.4
C01	415.8	425.0
C12	410.3	420.2
C20	426.6	435.9
C42	445.3	447.1
D27	456.1	459.4
D38	442.3	448.9
E38	375.3	380.4
E40	381.3	384.5
F14	381.8	389.2
F21	420.9	423.5
F25	396.7	407.9
F32	692.0	695.8
G11	416.4	418.3
G23	420.6	426.2
I09	507.9	520.7
I20	403.5	407.8
I24	410.1	408.8
I25	445.8	457.9

*) measured 2003, **) measured 2004

5 Conclusions

The calorimetric measurement method used is working as anticipated and has, up to now, shown good reproducibility. The estimated uncertainty of the measurement equipment is less than 1% for BWR and less than 2% for PWR (one standard deviation) based on repeated measurements on the same assembly.

The measured decay heat values have been compared to calculated values. The comparison shows good agreement for BWR. For PWR the calculations seem to over predict the decay heat with 2%.

References

- /1/ Rapport angående avgiven gammaeffekt i BWR- och PWR-bränsle. Institutionen för strålningsvetenskap Uppsala reg.nr. 2004-05604.
- /2/ NUREG/CR-0200, Revision 7, SAS2: A COUPLED ONE-DIMENSIONAL DEPLETION AND SHIELDING ANALYSIS MODULE, I. C. Gauld, O. W. Hermann, Date Published: May 2004.

Power and temperature values in the calibration curve

Startdate	Power (W)	Pool temp (°C)	dt/dT at T=0°C	Box/no box
30508	49	19.2	8.11E-05	box
30408	52	18.5	8.18E-05	
30210	55	18.3	8.40E-05	
30314	97	18.0	1.08E-04	
30327	98	18.4	1.08E-04	
30512	98	19.6	1.09E-04	box
30528	98	19.8	1.08E-04	box
30428	99	19.0	1.09E-04	
40202	143	19.9	1.34E-04	box?
21216	145	18.8	1.36E-04	
40203	146	20.0	1.37E-04	box
30107	147	19.1	1.38E-04	
30321	147	18.2	1.37E-04	
30325	147	18.3	1.35E-04	
31103	147	19.1	1.38E-04	box
30430	147	19.1	1.39E-04	box
30526	147	19.8	1.35E-04	box
30407	149	18.5	1.38E-04	
30516	193	19.6	1.65E-04	box
30422	195	18.9	1.65E-04	
30220	197	17.9	1.67E-04	
40204	202	19.9	1.69E-04	box
40205	202	19.9	1.69E-04	box
30411	245	18.6	1.94E-04	
30319	246	18.2	1.94E-04	
30520	247	19.8	1.93E-04	box
30502	295	19.1	2.25E-04	box
30131	297	18.6	2.25E-04	
30324	337	18.2	2.46E-04	
31104	343	19.1	2.49E-04	box
30521	349	19.8	2.53E-04	box
30519	392	19.2	2.77E-04	box
30320	395	18.2	2.80E-04	
30114	441	19.1	3.07E-04	
30522	445	19.9	3.08E-04	box
30321	491	18.2	3.36E-04	
30601	493	19.7	3.38E-04	box
30505	493	19.1	3.41E-04	box
30523	535	19.8	3.61E-04	box
30524	539	19.8	3.64E-04	box

Startdate	Power (W)	Pool temp (°C)	dt/dT at T=0°C	Box/no box
30327	545	18.4	3.67E-04	
30423	588	18.9	3.93E-04	
30326	589	18.3	3.92E-04	
30124	597	18.8	3.96E-04	
30318	687	18.1	4.56E-04	
30325	687	18.3	4.48E-04	
30424	689	18.9	4.49E-04	
30328	731	18.4	4.78E-04	
30506	735	19.1	4.83E-04	box
30317	781	18.0	5.05E-04	
30324	884	18.2	5.63E-04	

F-factors for gamma escape-calculations

Assembly no	F-factor	Assembly no	F-factor
2014	0.976	8327	0.951
2018	0.915	8331	0.987
2048	0.963	8332	1.004
2074	0.956	8338	0.939
2118	0.881	8341	0.980
9329	0.994	8327	0.978
10288	1.036	6423	0.968
14076	0.970	6478	0.752
3838	1.038	A05	0.993
KU0100	1.012	A11	0.973
KU0269	0.897	C01	1.100
KU0278	0.969	C12	1.011
KU0282	0.937	C20	1.031
5535	1.146	C42	1.047
11494	1.052	D27	0.960
11495	1.022	D38	1.002
13775	1.014	E38	0.961
13847	1.007	E40	1.006
13848	1.001	F14	0.999
12078	0.995	F21	0.972
13628	1.025	F25	1.020
13630	1.002	F32	1.029
1377	1.081	G11	0.995
1389	0.880	G23	0.994
1546	0.989	I09	1.059
1696	1.015	I20	0.956
1704	0.955	I24	1.055
2995	0.946	I25	0.975
3054	1.007	E38	0.985
3058	0.990	2A5	1.031
3064	0.969	5A3	1.019
6350	1.011	0C9	0.977
12684	0.986	1C2	1.033
582	0.991	1C5	0.999
596	1.014	2C2	1.004
710	0.982	3C1	1.050
900	0.995	3C4	1.098
1136	1.010	3C5	1.025
1177	1.015	3C9	0.942
1186	0.969	4C4	1.001
5829	0.987	4C7	1.023
6423	0.965	0E5	1.032
6432	0.965	0E6	1.028
6454	0.913	1E5	1.036
6478	0.920	5F2	0.976

Correction factors to compensate for thermal capacity difference between electric heater and BWR fuel assemblies

No	Volume dm ³	Zirconium kg	SS/lnc kg	UO2 kg	Correction factor
2014	36.2	84.5	9.2	204.2	0.963
2018	36.2	84.5	9.2	204.1	0.963
2048	36.2	84.5	9.2	204.0	0.963
2074	36.2	84.5	9.2	203.9	0.962
2118	36.2	84.5	9.2	203.9	0.962
9329	36.2	84.5	9.2	202.9	0.962
10288	36.2	82.9	9.2	203.4	0.962
14076	36.2	84.5	9.2	203.8	0.962
3838	35.8	82.2	9.3	201.9	0.963
KU0100	36.3	84.3	9.3	198.5	0.961
KU0269	36.2	81.9	9.3	200.9	0.962
KU0278	36.2	81.9	9.3	201.1	0.962
KU0282	36.2	81.9	9.3	201.0	0.962
5535	35.8	82.2	9.2	201.7	0.963
11494	36.4	77.1	9.2	205.5	0.961
11495	36.4	77.1	9.2	205.5	0.961
13775	36.4	77.1	9.2	205.8	0.961
13847	35.5	75.2	13.0	205.1	0.964
13848	35.5	75.2	13.0	205.1	0.964
1377	36.5	84.7	9.2	208.4	0.962
1389	36.5	84.7	9.2	208.5	0.962
1546	36.5	84.7	9.2	208.8	0.962
1696	36.5	84.7	9.2	209.1	0.962
1704	36.5	84.7	9.2	208.9	0.962
2995	36.5	84.7	9.2	203.6	0.962
3054	33.2	79.4	9.2	182.0	0.966
3058	33.2	79.4	9.2	182.0	0.966
3064	33.2	79.4	9.2	182.0	0.966
6350	36.1	82.3	9.2	203.2	0.962
12684	36.4	76.9	13.0	206.9	0.962
12078	36.3	82.3	9.3	201.3	0.961
13628	35.6	75.3	13.0	205.2	0.964
13630	35.6	75.3	13.0	205.2	0.964
582	29.5	47.0	4.2	201.4	0.971
596	29.5	47.0	4.2	201.1	0.970
710	29.5	47.0	4.2	201.3	0.970
900	29.5	47.0	4.2	201.3	0.970
1136	28.4	47.0	4.2	194.8	0.972
1177	35.0	79.1	9.2	205.0	0.964

No	Volume dm³	Zirconium kg	SS/Inc kg	UO2 kg	Correction factor
1186	35.0	79.1	9.2	204.9	0.964
5829	33.1	82.5	9.2	177.5	0.966
6423	29.0	48.0	4.2	201.7	0.972
6432	29.0	48.0	4.2	201.5	0.972
6454	29.0	48.0	4.2	201.7	0.972
6478	22.2	48.0	4.2	143.8	0.981
8327	29.0	48.0	4.2	201.5	0.972
8331	29.0	48.0	4.2	201.7	0.972
8332	29.0	48.0	4.2	201.5	0.972
8338	29.0	48.0	4.2	201.6	0.972
8341	29.0	48.0	4.2	200.3	0.972

Correction factors to compensate for thermal capacity difference between electric heater and PWR fuel assemblies

No	Volume dm ³	Zirconium kg	SS/lnc kg	UO2 kg	Correction factor
A05	75.2	111.3	18.1	518.3	0.916
A11	75.2	111.3	18.1	519.1	0.917
C01	75.2	111.3	18.1	517.4	0.916
C12	75.2	111.3	18.1	515.0	0.916
C20	75.2	111.3	18.1	516.2	0.916
C42	75.2	111.5	18.1	515.2	0.916
D27	75.5	129.1	17.6	491.0	0.915
D38	75.5	129.1	17.6	492.9	0.915
E38	75.5	129.1	17.6	492.2	0.915
E40	75.5	129.1	17.6	492.9	0.915
F14	75.5	129.1	17.6	495.3	0.915
F21	75.5	129.1	17.6	494.8	0.915
F25	75.5	129.1	17.6	496.4	0.915
F32	75.5	129.1	17.6	496.0	0.915
G11	75.5	129.1	17.6	495.1	0.915
G23	75.5	129.1	17.6	495.0	0.915
I09	75.5	129.1	17.6	496.4	0.915
I20	74.8	128.0	17.6	481.2	0.915
I24	75.5	129.1	17.6	487.6	0.914
I25	75.5	129.1	17.6	491.6	0.915
2A5	76.7	117.0	20.9	524.4	0.915
5A3	76.7	117.0	20.9	523.8	0.915
0C9	76.7	117.0	20.9	519.5	0.915
1C2	76.7	117.0	20.9	521.1	0.915
1C5	76.7	117.0	20.9	519.9	0.915
2C2	76.7	117.0	20.9	521.6	0.915
3C1	76.7	117.0	20.9	520.4	0.915
3C4	76.4	116.6	20.9	517.8	0.915
3C5	76.7	117.0	20.9	520.9	0.915
3C9	76.7	117.0	20.9	521.2	0.915
4C4	76.7	117.0	20.9	521.1	0.915
4C7	76.7	117.0	20.9	520.2	0.915
0E2	76.7	117.2	20.9	526.2	0.916
0E6	76.7	117.2	20.9	524.1	0.915
1E5	76.7	117.2	20.9	526.6	0.916
5F2	74.4	126.5	14.9	512.8	0.919

Raw data from the measurements BWR

Assembly no	Measured date	Pool temp (°C)	Slope value	Corrected slope value	Decay power (W)
9329	30602	19.7	1.85E-04	1.78E-04	217
9329	30604	19.7	1.86E-04	1.79E-04	219
6432	30610	19.8	1.61E-04	1.56E-04	180
6432	30612	19.8	1.63E-04	1.58E-04	184
11495	31107	19.1	1.52E-04	1.46E-04	163
1186	31110	19.1	1.36E-04	1.32E-04	137
2018	31111	19.1	1.03E-04	9.96E-05	82
10288	31112	19.1	1.63E-04	1.57E-04	181
13847	31113	19.1	1.53E-04	1.48E-04	165
13847	31114	19.1	1.53E-04	1.47E-04	165
2074	31117	19.2	1.11E-04	1.07E-04	95
12078	31118	19.2	1.24E-04	1.20E-04	117
2048	31119	19.3	1.10E-04	1.05E-04	92
9329	31120	19.4	1.82E-04	1.75E-04	213
13848	31124	19.2	1.53E-04	1.48E-04	166
2014	31125	19.1	1.03E-04	9.87E-05	81
1177	31126	19.2	1.58E-04	1.52E-04	173
1389	31128	19.2	1.03E-04	9.93E-05	82
KU0282	31201	19.1	1.82E-04	1.75E-04	213
11494	31202	19.2	1.51E-04	1.45E-04	162
1696	31203	19.2	1.08E-04	1.04E-04	90
6350	31204	19.2	1.28E-04	1.24E-04	124
6350	31205	19.2	1.30E-04	1.25E-04	126
1704	31208	19.0	1.03E-04	9.93E-05	82
14076	31209	19.1	1.94E-04	1.87E-04	234
3838	31210	19.3	1.28E-04	1.23E-04	123
3838	31211	19.3	1.28E-04	1.23E-04	123
13630	31212	19.3	1.91E-04	1.84E-04	229
3058	31215	19.3	1.28E-04	1.23E-04	123
12684	31216	19.3	2.19E-04	2.11E-04	275
2118	31217	19.3	1.12E-04	1.07E-04	96
5535	31218	19.4	1.04E-04	9.97E-05	82
13775	31219	19.4	1.58E-04	1.52E-04	173
KU0278	31222	19.4	1.68E-04	1.62E-04	190
2995	40107	19.2	1.31E-04	1.26E-04	127
13628	40108	19.2	1.67E-04	1.61E-04	188
KU0100	40109	19.2	1.62E-04	1.56E-04	180
5829	40112	19.3	1.77E-04	1.71E-04	205
3064	40113	19.3	1.25E-04	1.21E-04	119
3054	40114	19.4	1.36E-04	1.31E-04	137
1546	40116	19.4	1.17E-04	1.13E-04	105

Assembly no	Measured date	Pool temp (°C)	Slope value	Corrected slope value	Decay power (W)
KU0269	40119	19.4	1.67E-04	1.60E-04	187
1377	40122	19.6	8.70E-05	8.37E-05	55
710	40206	19.9	1.09E-04	1.06E-04	93
1136	40209	19.8	1.09E-04	1.06E-04	93
1136	40210	19.7	1.09E-04	1.06E-04	93
582	40211	19.7	1.07E-04	1.04E-04	89
6423	40212	19.6	1.54E-04	1.50E-04	169
6432	40213	19.6	1.60E-04	1.55E-04	179
6478	40216	19.6	1.22E-04	1.20E-04	118
6454	40217	19.7	1.61E-04	1.57E-04	181
900	40218	19.8	1.09E-04	1.06E-04	94
596	40219	19.9	1.04E-04	1.01E-04	85
596	40220	20.0	1.06E-04	1.03E-04	88
6432	40223	20.0	1.59E-04	1.54E-04	177
6432	40224	20.1	1.60E-04	1.56E-04	179
6432	40301	20.5	1.58E-04	1.54E-04	176
6432	40302	20.6	1.59E-04	1.54E-04	176
8332	40304	20.8	1.50E-04	1.46E-04	163
8338	40309	20.9	1.52E-04	1.47E-04	165
8327	40310	20.9	1.67E-04	1.62E-04	191
8331	40311	20.8	1.61E-04	1.57E-04	181
8341	40312	20.7	1.49E-04	1.45E-04	160
8341	40315	20.5	1.48E-04	1.43E-04	158
710	40322	20.5	1.07E-04	1.04E-04	90
710	40323	20.7	1.07E-04	1.04E-04	89

Raw data from the measurements PWR

Assembly no	Measured date	Pool temp (°C)	Slope value	Corrected slope value	Decay power (W)
5A3	30613	19.8	2.03E-04	1.85E-04	232
5A3	30616	19.7	2.02E-04	1.85E-04	231
5A3	30618	19.7	2.06E-04	1.89E-04	238
E38	40330	20.9	2.89E-04	2.64E-04	367
E38	40331	20.9	2.87E-04	2.63E-04	365
c42	40401	20.8	3.29E-04	3.02E-04	432
c42	40402	20.6	3.33E-04	3.05E-04	438
D38	40405	20.4	3.30E-04	3.02E-04	432
C12	40406	20.4	3.10E-04	2.84E-04	402
I25	40413	20.2	3.31E-04	3.03E-04	434
C20	40414	20.5	3.13E-04	2.87E-04	406
C20	40415	20.5	3.20E-04	2.93E-04	416
C20	40416	20.5	3.21E-04	2.94E-04	419
C20	40423	20.6	3.26E-04	2.98E-04	426
D27	40426	20.4	3.38E-04	3.09E-04	445
5A3	40427	20.4	1.98E-04	1.81E-04	225
3c5	40428	20.4	3.66E-04	3.35E-04	489
3C4	40429	20.4	3.64E-04	3.33E-04	485
3C1	40430	20.4	3.47E-04	3.17E-04	458
2A5	40503	20.3	2.00E-04	1.83E-04	228
4C7	40504	20.4	3.64E-04	3.33E-04	486
2C2	40505	20.6	3.45E-04	3.15E-04	455
0C9	40506	20.7	3.60E-04	3.29E-04	479
3C9	40507	20.8	3.46E-04	3.16E-04	456
5A3	40510	20.9	1.98E-04	1.81E-04	224
I20	40513	20.9	3.06E-04	2.80E-04	394
IC2	40514	20.9	3.14E-04	2.88E-04	407
I09	40517	21.0	3.70E-04	3.38E-04	495
G23	40518	21.0	3.16E-04	2.89E-04	410
5F2	40519	21.0	4.95E-04	4.55E-04	696
G11	40524	21.0	3.13E-04	2.87E-04	406
I24	40526	21.2	3.09E-04	2.82E-04	398
4C4	40527	21.1	3.17E-04	2.90E-04	411
F32	40528	21.2	4.85E-04	4.44E-04	675
C01	40610	21.3	3.13E-04	2.87E-04	406
1E5	40611	21.3	3.46E-04	3.16E-04	457
E40	40614	21.2	2.92E-04	2.67E-04	372
0E2	40616	21.4	4.19E-04	3.84E-04	572
1C5	40617	21.3	3.65E-04	3.34E-04	487
F14	40618	21.3	2.92E-04	2.67E-04	372
F25	40621	21.6	3.01E-04	2.76E-04	387
0E6	40622	21.7	3.57E-04	3.27E-04	475
F21	40623	21.8	3.16E-04	2.89E-04	410

Raw data from gamma measurements

Fuel assembly	Measurement date	Probe 1 Gy/h	Probe 2 Gy/h	Probe 3 Gy/h	Probe 4 Gy/h	Probe 5	Gamma escape power Gy/h(W)
11495	31107	9.5	3.8	1.3	0.5	0.3	4.3
1186	31110	7.7	3.1	1.0	0.4	0.2	3.3
2018	31111	4.9	2.0	0.6	0.2	0.1	2.0
10288	31112	10.1	4.0	1.4	0.5	0.3	4.6
13847	31113	10.9	4.4	1.5	0.6	0.3	4.9
13847	31114	11.0	4.4	1.5	0.6	0.3	4.9
2074	31117	5.5	2.2	0.6	0.2	0.1	2.3
12078	31118	7.4	3.0	1.0	0.3	0.2	3.3
2048	31119	5.3	2.1	0.6	0.2	0.1	2.2
9329	31120	12.3	4.9	1.7	0.6	0.4	5.4
13848	31124	11.4	4.5	1.6	0.6	0.3	5.1
2014	31125	4.4	1.7	0.5	0.1	0.1	1.9
1177	31126	9.7	3.9	1.3	0.5	0.3	4.5
1389	31128	5.6	2.2	0.7	0.2	0.1	2.2
KU0282	31201	14.3	5.7	2.1	0.7	0.4	5.9
11494	31202	10.0	4.0	1.4	0.5	0.3	4.4
1696	31203	5.2	2.1	0.6	0.2	0.1	2.3
6350	31204	7.4	2.9	1.0	0.3	0.2	3.2
1704	31208	5.1	2.0	0.6	0.2	0.1	2.1
14076	31209	15.3	6.1	2.3	0.8	0.5	6.6
3838	31210	7.3	3.0	1.1	0.3	0.2	3.4
3838	31211	7.3	3.0	1.0	0.3	0.2	3.4
13630	31212	16.0	6.3	2.4	0.8	0.5	7.1
3058	31215	7.9	3.2	1.1	0.3	0.2	3.5
12684	31216	17.9	7.1	2.7	1.0	0.5	7.8
2118	31217	6.1	2.5	0.8	0.2	0.2	2.4
5535	31218	4.3	1.8	0.5	0.1	0.1	2.2
13775	31219	11.8	4.6	1.7	0.6	0.3	5.3
KU0278	31222	12.9	5.1	1.9	0.7	0.4	5.5
2995	40107	7.7	3.0	1.0	0.3	0.2	3.2
13628	40108	12.8	5.1	1.9	0.7	0.4	5.8
KU0100	40109	11.5	4.6	1.7	0.6	0.3	5.1
5829	40112	12.6	5.0	1.8	0.6	0.3	5.5
3064	40113	7.3	3.0	1.0	0.3	0.2	3.2
3054	40114	8.2	3.3	1.2	0.4	0.2	3.7
1546	40116	6.2	2.5	0.8	0.2	0.1	2.7
KU0269	40119	14.0	5.5	2.0	0.7	0.4	5.6
1377	40122	3.0	1.3	0.4	0.0	0.1	1.4
1136	40209	5.9	2.3	0.7	0.2	0.1	2.6
1136	40210	5.9	2.3	0.7	0.2	0.1	2.6
582	40211	5.2	2.1	0.7	0.2	0.1	2.3

Fuel assembly	Measurement date	Probe 1 Gy/h	Probe 2 Gy/h	Probe 3 Gy/h	Probe 4 Gy/h	Probe 5	Gamma escape power Gy/h(W)
6423	40212	12.1	4.7	1.7	0.6	0.3	5.2
6432	40213	13.0	5.0	1.8	0.6	0.3	5.5
6478	40216	11.3	4.3	1.5	0.5	0.3	3.7
6454	40217	12.5	4.9	1.7	0.6	0.3	5.0
900	40218	6.4	2.6	0.8	0.2	0.1	2.8
596	40219	6.6	2.6	0.8	0.2	0.1	3.0
596	40220	6.6	2.6	0.8	0.2	0.1	3.0
6432	40223	13.4	5.2	1.9	0.7	0.4	5.7
6432	40224	13.5	5.2	1.9	0.6	0.4	5.7
6432	40301	13.6	5.2	1.9	0.7	0.3	5.8
6432	40302	13.0	5.0	1.8	0.6	0.3	5.5
8332	40304	11.6	4.5	1.6	0.5	0.3	5.1
8338	40309	11.2	4.4	1.5	0.5	0.3	4.6
8327	40310	14.1	5.5	1.9	0.7	0.3	6.0
8331	40311	13.2	5.1	1.8	0.6	0.3	5.7
8341	40312	11.1	4.3	1.5	0.5	0.3	4.8
8341	40315	11.1	4.3	1.5	0.5	0.3	4.8
710	40322	6.9	2.8	0.8	0.2	0.1	3.0
710	40323	7.0	2.8	0.9	0.2	0.1	3.0
E38	40330	21.5	8.0	2.7	1.0	0.5	9.4
E38	40331	21.6	8.1	2.8	1.0	0.5	9.4
C42	40401	22.2	8.4	2.9	1.1	0.5	10.6
C42	40402	22.7	8.6	3.0	1.1	0.6	10.8
D38	40405	23.6	8.9	3.2	1.2	0.6	10.8
C12	40406	18.9	7.3	2.5	0.9	0.5	8.7
I25	40413	26.7	10.1	3.5	1.3	0.6	11.8
C20	40414	20.6	7.9	2.7	1.0	0.5	9.7
C20	40415	20.6	7.8	2.7	1.0	0.4	9.6
C20	40416	20.6	7.9	2.7	1.0	0.5	9.7
C20	40423	20.7	8.1	2.8	1.0	0.5	9.8
D27	40426	26.1	10.0	3.5	1.3	0.6	11.4
5A3	40427	12.5	4.9	1.5	0.5	0.2	5.8

Gamma scanning of spent nuclear fuel assemblies: determination of residual thermal power using new data acquisition and analysis software

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Abstract

New data-acquisition software has been developed for the gamma scanning of spent nuclear fuel assemblies. The system has been tested in two measurement campaigns at the central interim storage for spent nuclear fuel (Clab) in Oskarshamn Sweden.

The results from the measurement campaigns were used to determine calibration constants that relate the intensity of the 662 keV gamma radiation to the residual thermal power of the fuel. Using the calibration constants, the residual thermal power for each assembly in the data set was predicted. The predicted values were found to agree with the calorimetric values with a standard deviation of 2.23%.

Contents

1	Introduction	53
2	The method	55
3	Experimental equipment and instrumentation	59
3.1	The mechanical equipment	59
3.2	The gamma-ray detector	60
3.3	Data-acquisition system	60
	3.3.1 The hardware	60
	3.3.2 The software	61
3.4	Analysis procedure	63
4	Measurements	67
4.1	Fuel assemblies included in the assay	67
4.2	Gamma spectroscopic measurements	68
4.3	Calorimetric measurements	68
5	Analysis	69
5.1	Dead-time correction	69
5.2	Computing the relative intensities in selected gamma-ray lines	70
6	Experimental results	73
6.1	Correlation between the spectroscopic and the calorimetric measurements	73
7	Summary	79
8	Outlook	81
9	References	83
	Appendix A Fuel assemblies included in the assay	85
	Appendix B Calibration curves	89
	Appendix C Results from the calorimetric measurements	93
	Appendix D Summary of the measurement results	95

1 Introduction

The determination of the residual power in spent nuclear fuel assemblies is a necessary condition for the realization of the Swedish strategy for long-term storage of the spent nuclear fuel. The method of storage envisioned implies the encapsulation of fuel assemblies in copper canisters after a cooling time of about 30 years. Eventually these canisters will be embedded in bentonite clay in a deep geological storage. To maintain the buffer properties of the bentonite clay, an upper limit of 100°C [1] for the temperature on the surface of the copper canisters has been defined. Thus, knowledge of the thermal power developed in the spent fuel is essential for a safe storage.

The residual power may be calculated by using the various codes available, e.g. Origen. These codes generally calculate residual power with typical accuracies ranging between 2% and 5% [2]. However, the accuracy of the output is strongly correlated to the authenticity of the input data, e.g. irradiation history. From an operational point of view, such sensitivity is not desirable and should be relaxed by using complementary techniques such as measurements.

The measuring technique available until now is calorimetry [3] and [4]. Although this technique may be performed with high accuracy, it suffers from measuring times of the order of days per assembly. Such a long measuring time is not feasible for industrial application and an alternative method has therefore been proposed [5, 6]. This method is based on gamma-spectroscopic measurements performed by using gamma-scanning equipment of the type installed in all nuclear power plants and at the interim storage Clab in Sweden. As shown in [5, 6, 7, 8], the intensity from ^{137}Cs is closely related to the residual thermal power and this fact can be utilised for a fast and reasonably accurate determination of the residual power. In this work, it was shown that typical measuring and analysis times per assembly were about 15 minutes using the gamma scanning method, while the relative uncertainty in the determined residual power was about 2%.

In principle, the method is not dependent on that operator declared fuel parameters such as burnup and cooling time are available. In fact, these parameters may be experimentally determined by making use of the gamma-ray intensities from the decays of the isotopes: ^{137}Cs , ^{134}Cs and ^{154}Eu . This property makes the method interesting also from a safeguards point of view.

The gamma scanning technique thus provides a complement to pure theoretical approaches for two reasons; Firstly, complex calculations, in which mistakes can be made with regards to both the input parameters as well as the interpretation of the output, can be checked with a simple calculation based on experimentally obtained gamma intensities. Secondly, in cases where input data is questionable or lacking, a value of the residual power may still be obtained although with a somewhat larger uncertainty.

The present report accounts for a new data acquisition system equipped with special designed software. A dedicated calorimeter has been utilised for calibration of the gamma-scanning measurements using the acquisition system and the results of this work are also presented.

2 The method

The radiation from ^{137}Cs is suitable to use for several reasons:

- i) Studies using Origen-2 (/7/) showed that the intensity of ^{137}Cs is linearly dependent on the fuel burnup and nearly independent of such parameters as initial enrichment and void (in the case of BWR).
- ii) ^{137}Cs has a half-life of 30 years, which is adequate for measurements in connection to encapsulation, where cooling times of the fuel assemblies are expected to be in the range of 15–50 years.
- iii) The gamma-ray energy interferes very little with other gamma-ray intensities from fission or activation products in the gamma-ray spectrum (see Figure 5-2).

The relationship between the residual thermal power (P_{tot} expressed in Watts) of the spent fuel and the intensity (I expressed in counts per second) of the 662 keV gamma line is given by /5, 6/:

$$P_{\text{tot}} = C \frac{I}{f} \quad (2-1)$$

The factor f represents the fractional power due to the decay of ^{137}Cs and the constant C is a calibration constant that relates the power to the measured intensity. The factor f is defined as:

$$f = \frac{P_{137}}{P_{\text{tot}}} \quad (2-2)$$

Where P_{137} is the power developed in the fuel due to the decay of ^{137}Cs .

Figure 2-1 shows the values of f as a function of the cooling time for a BWR 8×8 fuel assembly, as calculated by Origen-2. Figure 2-1 shows that in this case the fractional power (f) reaches a maximum when the cooling time is about 13 years. The time to reach this maximum value is a function of the discharge burnup of the fuel.

Figure 2-2 shows the dependency of f on the discharge burnup for cooling times of 16 years, 27 years and 32 years respectively. An interesting feature of Figures 2-1 and 2-2 is the relative constancy of the factor f for cooling times greater than 13 years. For a typical time of final reposition of 30 years and a burnup of 36 GWd/tU, the factor f varies $\pm 10\%$ for a variation in the cooling time of ± 10 years.

Similarly, for a cooling time of about 30 years, the change in f is about $\pm 5\%$ for a change in burnup of ± 10 GWd/tU. These relatively small changes and the simple dependency of f are additional arguments for the feasibility to use the ^{137}Cs -intensity to determine the residual power, i.e. one do not expect drastic variations due to, for example, irregular irradiation histories.

As the factor f is included in Equation 2-1 one would argue that since f is calculated with e.g. Origen-2, the same computational uncertainties are introduced as when the residual power is determined entirely by using calculations. However, due to the simple dependencies of the factor f as discussed above, this factor can be parameterised in the variables burnup and cooling time, which are to be experimentally determined anyway.

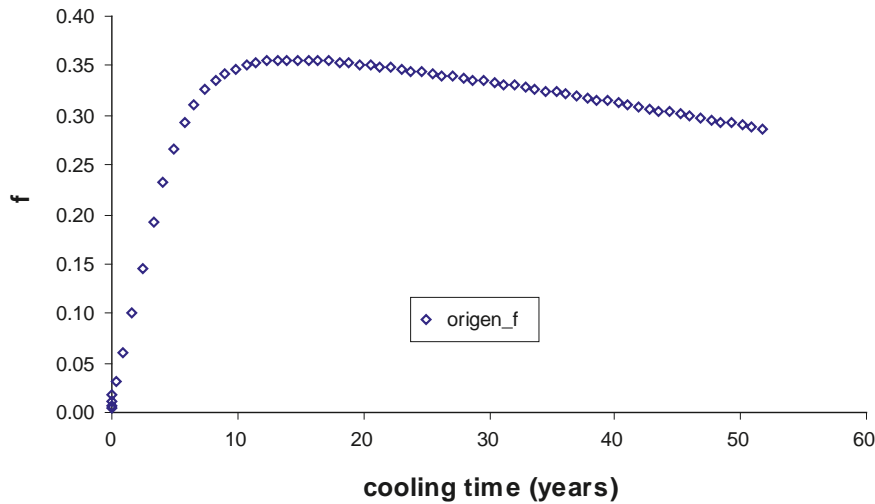


Figure 2-1. The value of the fractional power f as a function of the cooling time for a BWR 8×8 assembly irradiated for five power cycles. The discharge burnup of the fuel was 36.86 GWd/tU and the initial enrichment was 2.89% . The curve was plotted from Origen-2 data.

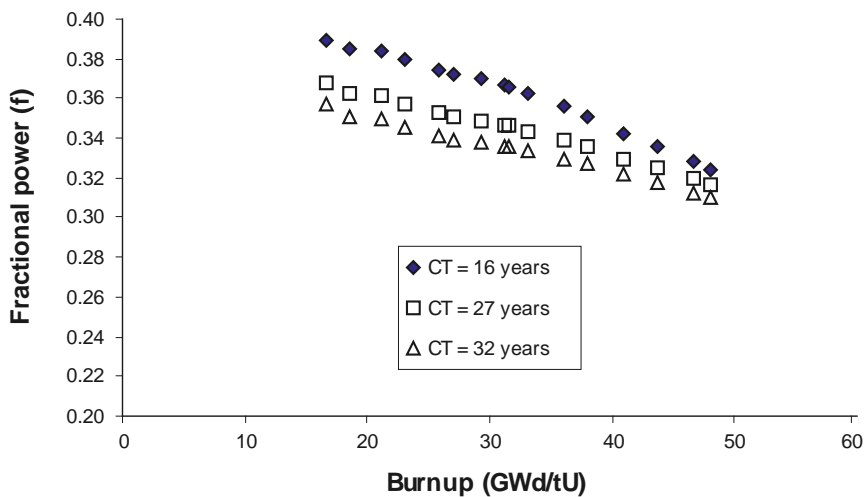


Figure 2-2. The dependency of the fractional power (f) on the discharge burnup for cooling times of 16, 27 and 32 years for a fuel assembly BWR 8×8 type, i.e. the same assembly type as in Figure 2-1.

Consequently these variables will be the only input to determine f and the experimentally determined residual power will thus not depend on detailed calculations where input parameters such as irradiation history may be erroneous. The basic requirement for this argument to hold is that f , as shown in figs. 2.1 and 2.2, represents reality with reasonable good accuracy. As will be shown in the later sections, this is indeed the case.

In this work we have adopted the following parameterization of f :

$$f = aBU + bT^2 + cT + d \quad (2-3)$$

i.e. a linear dependency on the discharge burnup (BU) and a quadratic dependency on cooling time (T). The latter dependency is motivated by the exponential decay of the radioactive isotopes. By expanding the exponential function to second order, the quadratic dependency of f on cooling time is obtained.

As the method includes relative measurements, it requires that the residual power is determined calorimetrically for a number of fuel assemblies with well-known properties in order to determine the calibration constant C in Equation 2-1. The value of C depends on the measuring geometry and the type of fuel assembly being investigated, implying that calibration curves have to be established for each fuel type of interest. Also, a representative assembly from each fuel type should be selected for regular efficiency calibration (since the measuring geometry may change between the measurements). Every new measuring campaign should start with measurements of these “reference” assemblies and the measured ^{137}Cs -intensities, properly corrected for decay time, would thus constitute a norm to which all other measurements within the same set of assembly type would be related.

3 Experimental equipment and instrumentation

The equipment and instruments used during the measuring campaigns discussed in Section 5 can be divided into three parts; the mechanical equipment (elevator and its associated control system), the gamma-ray detector and a computer-based data-acquisition and analysis system.

3.1 The mechanical equipment

The gamma scanning equipment consists of a collimator, an elevator, fixtures for holding the fuel assembly in a vertical position in the elevator and control mechanism for regulating the motion of the elevator, see Figure 3-1. The fuel assembly of interest is placed in the fixture and may be rotated about its symmetry axis. This allows each of the four corners of the assembly to be placed in such a way as to face the detector. In this way, the positioning uncertainty is minimised /9/ leading to maximum consistency of the measured gamma-ray intensities.

The collimator is made of iron with a length of 1.2 m. The collimator slit is horizontal and its height can be varied between 1 m and 5 mm. The width varies from 24.3 cm at the end facing the fuel assembly to 8.2 cm at the end facing the detector. This arrangement allows for a solid angle covering the diagonal of all fuel types of interest.

3.2 The gamma-ray detector

In this application, high-intensity gamma radiation is expected in conjunction with low-intensity radiation. This leads to the following requirements of the detector system:

1. The detector system should be able to record events at considerable count rates in order to reach sufficient statistics while minimizing the measuring time.
2. The detector should preferably be large in order to obtain high peak-to-Compton ratio and thereby facilitating accurate analysis of the energy spectra.

Based on the above requirements, an 80% germanium detector from Ortec, equipped with a transistor-reset preamplifier was chosen. This detector system allows for input count rates exceeding 100,000 counts per second (cps). The stated energy resolution was 2.0 keV and the peak-to-Compton ratio was 75:1. Both these parameters were measured at 1,322 keV. As a backup, a 55% detector from Eurisys Measures was available. The latter detector was however equipped with a resistive-feedback preamplifier, which limited the maximum count rate to about 60,000 counts per second. The backup detector was used in the second of the two measuring campaigns reported here.

3.3 Data-acquisition system

The data-acquisition software used in /5, 6, 7, 8/ was a test system that was highly hardware dependent. As new hardware is introduced in a very high pace, new software that adapt to this situation had to be developed. This section describes a data-acquisition system developed especially to collect and analyse data for the purpose of determining residual power.

3.3.1 The hardware

The output signals from the detector were connected to a series 5000 pc board from APTEC-NRC, which includes signal processing electronics and a multi-channel analyser. This board has three types of linear amplifiers, an ADC and an ADC memory. The system also contains user software supplied by the manufacturer and application programmers' interface (API) in the form of a dynamic linked library (DLL) of low-level functions. The API allows user the to write custom software that can be used to control and access the board in order to meet stipulated needs.

The on-board amplifiers are of the following types: unipolar, bipolar and gated integrator. Although giving a somewhat degraded energy resolution, the gated integrator is well suited for high-count-rate applications. However, the moderate count rates encountered in this work motivated the use of the standard unipolar amplifier with an integration time of $2\mu\text{s}$.

The analogue-to-digital conversion was made using a 12-bit ADC (4,096 channels) with a fixed conversion time of 800 ns. It converts the amplitude of the detector signal, i.e. the energy deposited in the detector, to a channel number or memory position. For each event recorded by the detector, the number stored in the corresponding memory position is incremented by one. In this way, an energy spectrum of the type shown in Figure 5-2 is obtained.

3.3.2 The software

Data-acquisition software was developed for use with the series 5000-board. This software acts as the interface between the user and the low-level i/o-functions that are used to access the series 5000-board. It allows the user to access the data stored in the ADC memory using the low-level i/o-functions while scanning an assembly. The number of times that the ADC memory is accessed, while scanning a corner of an assembly, is defined by the user. The spectrum collected and stored between each access to the ADC memory is called a sub-spectrum in this report.

Through the graphic user interface presented in Figure 4-3, the user can enter the necessary parameters for initialising the board and controlling the scanning procedure. The list below gives an example of the parameters that can be set by the user:

- The collection time for each sub-spectrum.
- The number of sub-spectra.
- Type of linear amplifier: The user can select one of the three linear amplifiers on board the MCA or make use of an external amplifier.
- Type of signal from the preamplifier.
- The amplifier gain.
- The amplifier time constant.
- Pole-zero adjustment.
- The type of preamplifier signal connected to the input of the board.
- The polarity of the input signal.
- The number of ADC channels to be used for collection (varies from 256 to 4,096 in powers of 2).
- Choice of pulse pile-up rejection/total live time correction: The user can choose to turn on or off the pulse pile up rejection circuit on the card.
- Pulse pile up discrimination level for the amplifier.

The software was designed in modules for easy adaptation to various hardware. The sub-spectrum files are stored in text format. Altogether this means that the analysis software is independent of any file format or hardware.

A flow chart of the spectrum collection and initialization module of the data-acquisition software is shown in the flow chart in Figure 3-2.

Dead-time losses due to accesses to the ADC memory are reduced to barest minimum using the timing and delay functions provided by the compiler system. The lost time (see flow chart) is usually in the range of 0.35 μ s. This time is subtracted from the collection time in order to set the delay time.

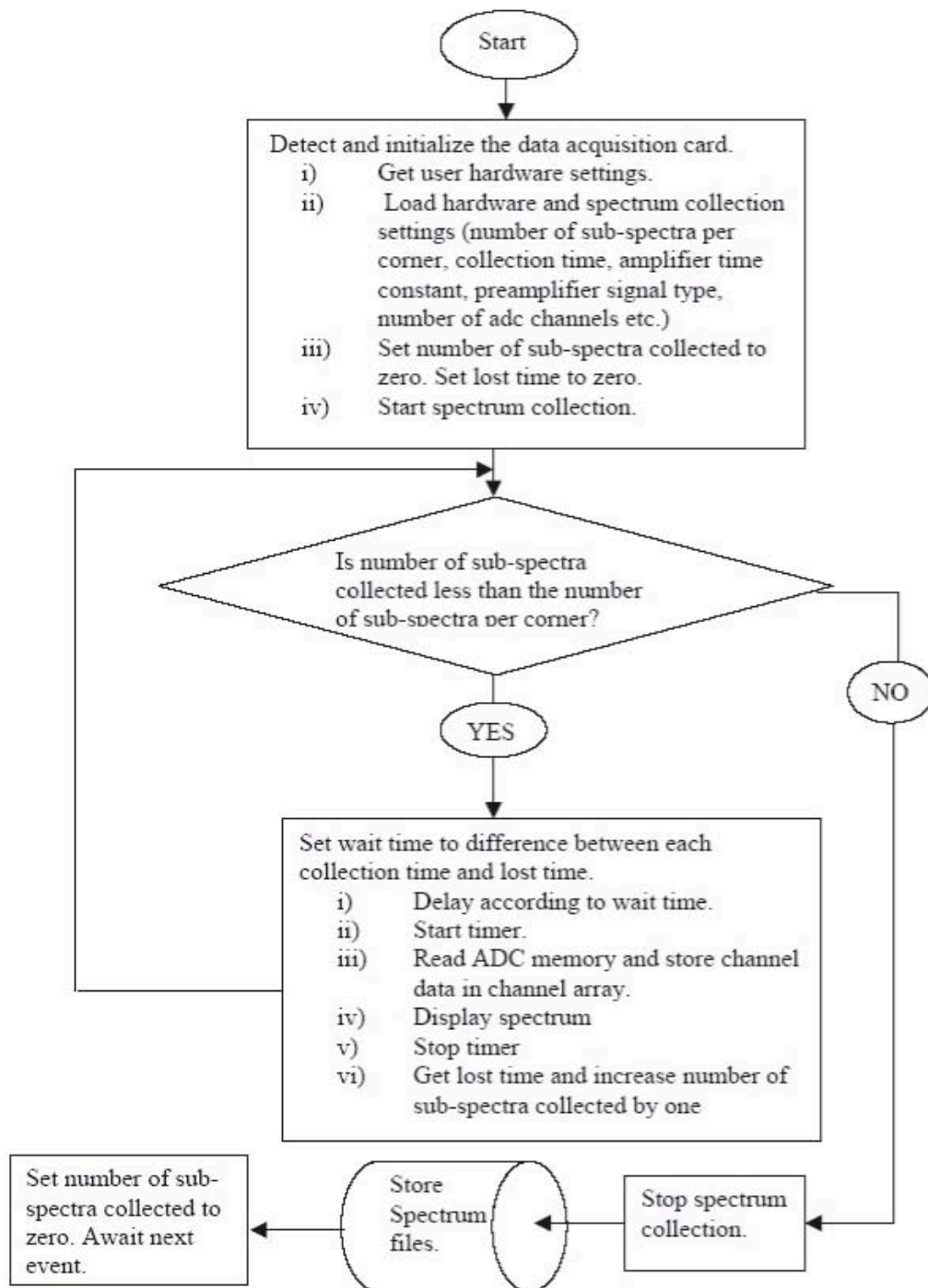


Figure 3-2. A flow chart showing the principles behind the data-acquisition software.

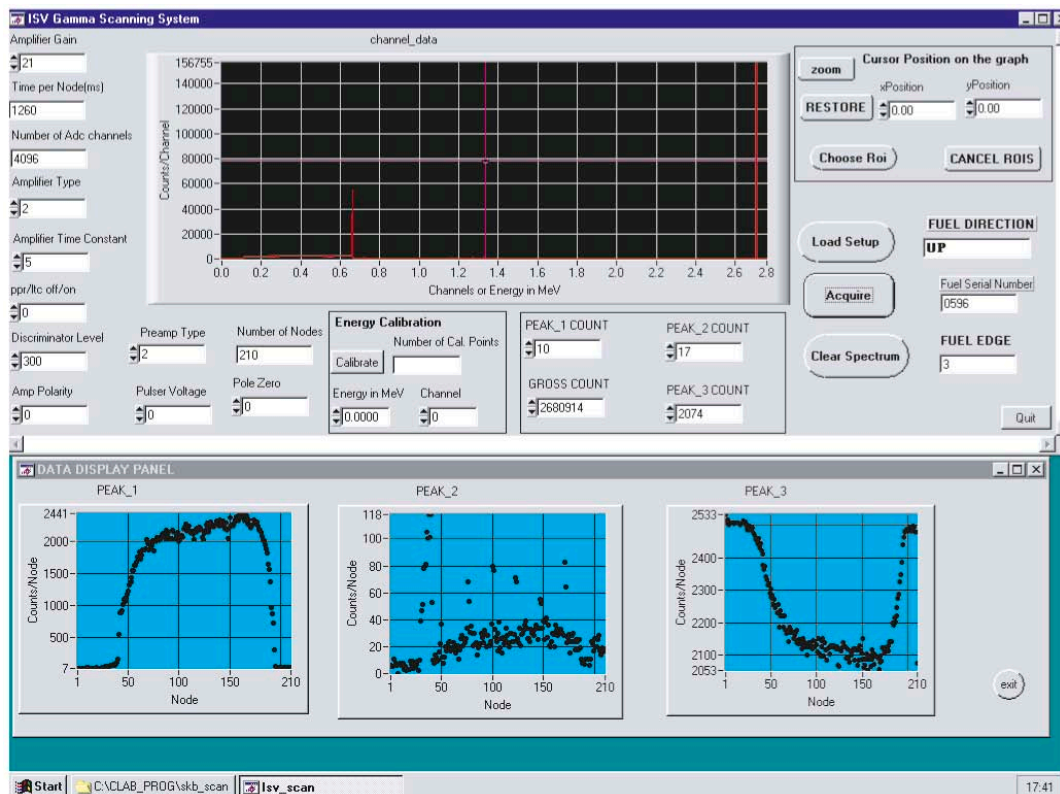


Figure 3-3. The graphic interface of the data-acquisition module, showing the gamma-ray spectrum and the profiles of three selected gamma-ray peaks. Here, peak_1 is the peak area for ^{137}Cs , which reflects the axial burnup profile, peak_2 is the peak area for ^{60}Co , which allows for detecting the positions of the spacers and peak_3 is the peak area for an artificial pulse peak used for obtaining the dead time profile. In this figure, a sub spectrum is called a node.

3.4 Analysis procedure

A software tool has been developed for analysing the data obtained in a measurement campaign. The tool is used to compute the relative intensities of ^{137}Cs for each fuel assembly by performing the following tasks:

- Dead-time correction of each sub spectrum.
- Computation of the relative intensities for each of three peaks in the spectrum.

It also performs the following tasks:

- i) Provision of diagnostic information about the analysis.
- ii) Verification of the discharge burnup and cooling time for each fuel assembly.
- iii) Computation of the calibration constant C in Equation 2-1.
- iv) Presentation of the residual thermal power using the constant C and the relative intensity of ^{137}Cs .

In order to use the analysis tool, the user is required to enter a file name where to find input information, such as the path to where the spectrum data files are stored, the date that each assembly was scanned, the fuel id, whether the assembly is reconstructed or not, fuel geometry, fuel type and enrichment. An example of the content of such an input file is shown in Figure 3-4.

Two other input files are required:

- i) Irradiation history file, which contains information about the core lifetime of each assembly, as declared by the power plant operators.
- ii) Residual thermal power file used for the calibration.

An example of the content of the irradiation history file is shown in Figure 3-5 and an example of the residual thermal power file is shown in Figure 3-6. The latter file also contains values of the fractional power (f) obtained from Origen-2.

Fuel id	Reactor type	Scan date	Path	Enrichment	Status	Fuel channel
1177	BWR	01-22-2003	C	2.65	OK	Yes
9329	BWR	01-22-2003	C	2.92	OK	Yes
1186	BWR	01-21-2003	C	2.65	OK	Yes
1389	BWR	01-23-2003	C	2.201	OK	Yes
1546	BWR	01-29-2003	C	2.201	OK	Yes
1696	BWR	01-23-2003	C	2.201	OK	Yes
1704	BWR	01-27-2003	C	2.201	OK	Yes
2014	BWR	01-22-2003	C	2.33	OK	Yes
2018	BWR	01-21-2003	C	2.33	OK	Yes
2048	BWR	01-21-2003	C	2.33	OK	Yes
2995	BWR	01-28-2003	C	2.699	OK	Yes
3058	BWR	01-28-2003	C	2.751	OK	Yes
3064	BWR	01-29-2003	C	2.751	OK	Yes
3838	BWR	01-28-2003	C	2.09	OK	Yes

Figure 3-4. An example of the contents of the input data file. The column named "Path" is used to determine the disk drive where the spectrum data are stored while the column named "Status" allows the analysis tool to determine if the fuel assembly has been rebuilt or not.

ID:C01		
06-19-1974	04-13-1977	11247
07-07-1977	03-31-1978	9403
05-26-1978	04-03-1979	7569
06-25-1979	04-01-1980	0
06-18-1980	04-04-1981	8469
06-23-1981	05-06-1982	0
07-29-1982	04-28-1983	0
07-28-1983	04-13-1984	0
07-12-1984	04-04-1985	0
06-14-1985	04-30-1986	0
07-02-1986	04-25-1987	0
06-18-1987	05-12-1988	0

Figure 3-5. An example showing the contents of the irradiation cycle file. The presented data accounts for the assembly with id number C01. The first column shows the date when a cycle was started, the second column shows the date the cycle ended while the third column shows the burnup (MWd/tU) of the fuel during the cycle. This file contains the data that is used for the verification of the operator's declared burn up and cooling time.

0582	8X8	02-11-2004	89.35	2.30	91.65	0.36532
0596	8X8	02-19-2004	85.33	2.96	88.29	0.36432
0596	8X8	02-20-2004	88.25	2.97	91.21	0.36432
0710	8X8	02-06-2004	93.32	3.01	96.33	0.36149
0710	8X8	03-22-2004	89.94	3.01	92.95	0.36149
0710	8X8	03-23-2004	89.29	3.02	92.31	0.36149
0900	8X8	02-18-2004	93.67	2.80	96.48	0.35940
1136	8X8	02-09-2004	92.61	2.62	95.23	0.35472
1136	8X8	02-10-2004	93.15	2.63	95.78	0.35472

Figure 3-6. An example showing the contents of the calibration information file. Each line gives the data for a given fuel assembly. The information in each column are: the fuel id, the fuel geometry type, the date of calorimetric measurement, the measured residual power, the estimated power due to the escape of gamma photons from the calorimeter, the total calorimetric power and the fractional power (f) at the time of the calorimetric measurements.

4 Measurements

In order to obtain the calibration constant C in Equation 2-1, data from calorimetric measurements and gamma scanning were used. These measurements are described below.

4.1 Fuel assemblies included in the assay

The measurements using the gamma scanning technique were performed during two campaigns: one in January 2003 and the other in June 2003. A total number of 86 assemblies from 10 reactors were scanned. 77 assemblies were scanned in January, 12 assemblies were scanned in June while 3 were scanned both in January and in June. A summary of the fuel assembly types scanned is shown in Table 4-1.

The initial enrichments of the assemblies varied from 1.95% to 3.43%. A full list of the scanned fuel assemblies is available in Appendix A.

Of the total 86 assemblies measured, 22 were excluded from the results and analysis for the following reasons:

- i) PWR fuel assemblies that contains control rods. The residual power of these assemblies was not measured with the calorimeter so they could not be used in the determination of the calibration constants.
- ii) Fuel assemblies that were rebuilt. These assemblies will be subject for further analysis.
- iii) Fuel assemblies of a type that were too few to be used in constructing a calibration curve.

The assemblies that were excluded are shown in Table A-3 Appendix A.

Table 4-1. The fuel types that were scanned in January and June 2003.

Reactor type	Total	Fuel type	Fuel channel	No of assemblies
BWR	50	8x8	Yes	24
		8x8	No	14
		SVEA64	Yes	4
		SVEA100	Yes	4
		KWU8x8	Yes	1
		KWU9x9-5	Yes	3
PWR	36	15x15	No	20
		17x17	No	16

4.2 Gamma spectroscopic measurements

To perform a scan, the assembly was placed in the fixture described in Section 3.1. The fixture was rotated so that the detector viewed a corner of the assembly. The scan was then performed by starting the data acquisition system and moving the assembly upwards in front of the collimator. When the scan was completed, the fixture was rotated 90° and the next scan was initiated by moving the assembly downwards.

The speed of the elevator was set to about 120 cm per minute while travelling upwards and about 150 cm per minute while travelling downwards. These speeds were considered a feasible compromise between short measuring times and small uncertainties due to counting statistics.

The different speeds were considered when setting the time windows used to collect and store data. When the elevator was travelling upwards, data was collected in each sub spectrum during a time of 1.26 seconds. When the elevator was travelling downwards, the measuring time was 1.01 seconds per sub spectrum. A total of 210 sub spectra per corner were collected.

The procedure was repeated for all four corners of the assembly. The measuring time per assembly was about 15 minutes. Including fuel handling, the measurements were performed at a typical rate of one fuel assembly every 30 minutes.

4.3 Calorimetric measurements

The calorimetric data were obtained from measurements performed by the staff of the Swedish Nuclear Fuel Company (SKB) and the OKG AB (parts 3 and 4). The same fuel assemblies that were used in the gamma scanning measurements were also used in the calorimetric measurements with the exception of two assemblies: A05 and A11. These two assemblies were excluded from the calorimetric measurements at this stage because they contain control rods. The results from the calorimetric measurements of the residual power are shown in Table C-1.

5 Analysis

5.1 Dead-time correction

In this work, gross input count rates reaching more than 130,000 cps were encountered. This together with the performance of the detector system implied dead-time losses that varied from a few percent up to about 70%. Such large varying dead-time losses necessitate a proper correction of the measured count rates. In this work, the stored sub spectra for each fuel assembly were corrected for dead-time using the pulser method. External pulses from a pulse generator with a well-defined event rate of $(2,000 \pm 1)$ Hz were injected into the detector system.

By forming the ratio between the number of events injected during the measuring time and the number of events in the pulser peak, a dead-time correction factor was obtained for each sub spectrum. The number of counts in each channel in a given sub spectrum was then multiplied with this factor to obtain a dead-time corrected spectrum.

Figure 5-1 shows the variation of the dead-time correction factor along the axis of the fuel assembly 5F2.

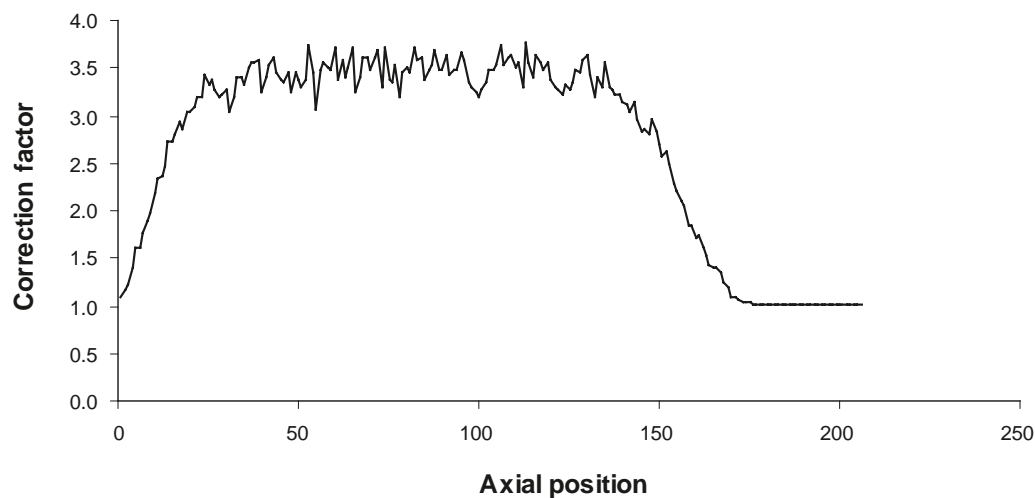


Figure 5-1. Variation of the dead-time correction factor along the axis of the fuel assembly with the assembly id: 5F2. The maximum gross count rate for a sub-spectrum during this particular measurement was about 42,000 counts per second (obtained from non dead-time corrected spectra). The fuel assembly had a discharge burnup of 47GWd/tU and a cooling time of about 12 years.

5.2 Computing the relative intensities in selected gamma-ray lines

For a given fuel assembly, the dead-time-corrected sub spectra for each of the four corners were summed up to give a total spectrum. In such a way, four energy spectra were obtained for further analysis to determine the net count rate in each selected gamma-ray peak. Figure 5-2 shows an example of such an energy spectrum.

The first step in this procedure was to locate the position of each selected gamma ray peak in the spectrum using a search algorithm. Here, the peaks indicated in Figure 5-2 were selected i.e. from ^{137}Cs , ^{134}Cs , and ^{154}Eu respectively. When a peak was found, its centroid and full width at half maximum (FWHM) were determined. The peak position and the FWHM were used to define windows that correspond to peak and background regions, which were used to calculate the net count rate in the peak, see Figure 5-3.

In this work, the windows selected for the gamma-ray peaks were 14 channels wide while the window selected for the pulser peak was 12 channels wide. The total number of counts in a peak was obtained by summing the counts in regions 4 and 5 in Figure 5-3. The regions 1 and 6 in the same figure defined the background regions so that the contents in regions 2 and 3 were subtracted from the total number of counts to obtain the net content in the peak. The details of the technique used to obtain the background contribution are described in /12, 13/.

The net content obtained for each selected peak was used to compute the corresponding relative intensity for that given peak from:

$$\text{Relative intensity } (I_r) = \frac{\text{Net count in the peak}}{\text{True time}} \quad (5-1)$$

True time is defined as the actual measuring time because the analyzed spectra were dead time corrected.

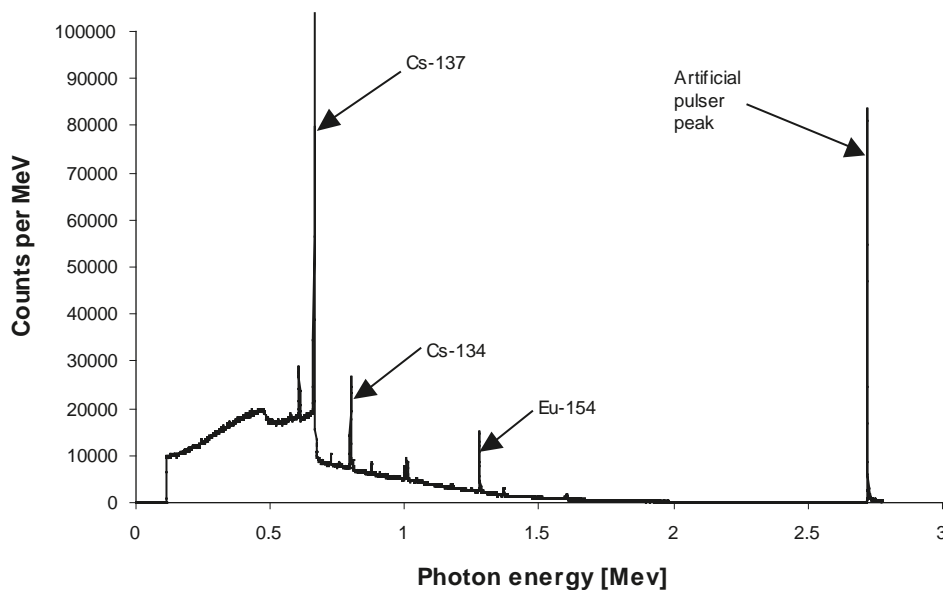


Figure 5-2. The gamma-ray energy spectrum collected from the fuel assembly: 5F2. The assembly had a cooling time of 11.6 years. The following spectrum lines are indicated: ^{137}Cs at 662 keV, ^{134}Cs at 794 keV, ^{154}Eu at 1,274 keV and the artificial pulser peak at about 2,700 keV.

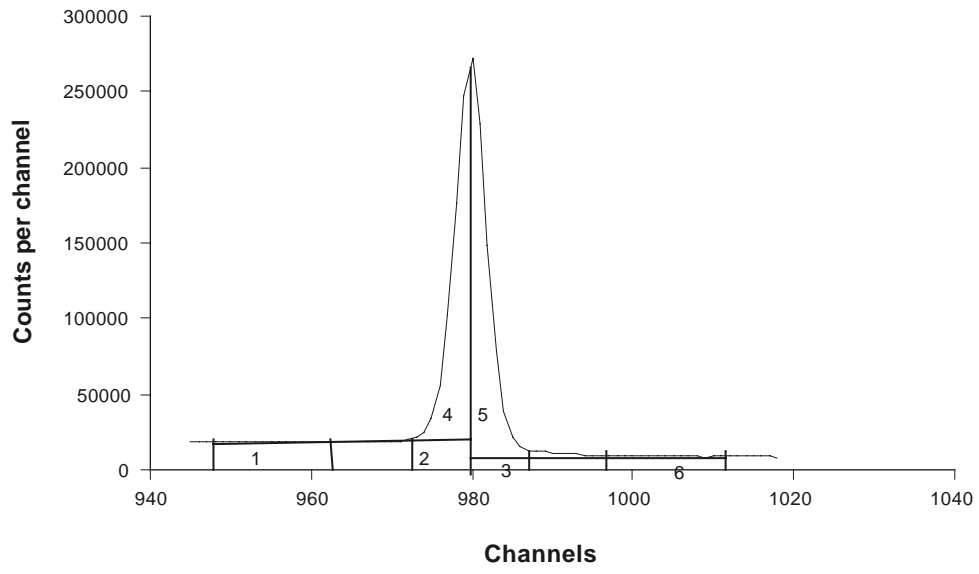


Figure 5-3. The windows defining the peak and the background areas.

6 Experimental results

To determine the residual heat using the gamma scanning technique, the measurements have to be calibrated. The calibration was done by using data obtained from calorimetric measurements.

6.1 Correlation between the spectroscopic and the calorimetric measurements

The value of the factor f for each assembly was obtained by using Equation 2-3 and 2-2. The values of the fitting constants a , b , c and d of Equation 2-3 are shown in Table 6-1. These fitting constants are independent of the fuel design.

From the gamma scanning measurements, the relative intensity I of the gamma radiation from the ^{137}Cs decay was obtained enabling the calculation of $\frac{I}{f}$ for each assembly. The results are shown in Table 6-2.

Table 6-1. Numerical values of the fitting coefficients in Equation 2-3. The values were determined using linear regression methods.

Fitting constant	Numerical value
a (tU/GWd)	-1.44×10^{-6}
b (/days ²)	-1.82×10^{-10}
c (/days)	-1.00×10^{-06}
d	4.17×10^{-1}

Table 6-2. The ratio $\frac{I}{f}$ for the fuel assemblies that were scanned and analysed in this study.

Fuel id	f	$\frac{I}{f}$ (cps)	Uncertainty of $\frac{I}{f}$ * (cps)	Fuel id	f	$\frac{I}{f}$ (cps)	Uncertainty of $\frac{I}{f}$ * (cps)
1177	0.350	5,445	386	3C1	0.350	14,872	1,052
9329	0.348	6,932	491	3C5	0.348	15,840	1,121
1186	0.359	4,328	307	3C9	0.350	14,791	1,046
1389	0.369	2,605	185	4C4	0.355	13,535	958
1546	0.365	3,416	242	4C7	0.348	15,928	1,127
1696	0.370	2,844	202	0E2	0.345	18,615	1,317
1704	0.370	2,578	183	0E6	0.354	15,202	1,075
2995	0.354	4,034	286	1E5	0.356	14,536	1,028
3838	0.373	3,700	262	1O9	0.347	16,576	1,173
6350	0.362	4,166	295	D27	0.342	14,929	1,056
10288	0.356	5,812	411	F25	0.348	12,686	898
12078	0.370	3,914	277	D38	0.341	14,509	1,026
14076	0.352	7,239	512	E38	0.349	12,431	879
8338	0.356	6,108	432	E38	0.349	12,532	887
8327	0.354	7,074	501	E40	0.348	12,214	864
1136	0.363	3,464	245	F14	0.350	12,319	872
6423	0.355	6,464	458	F21	0.348	13,416	949
6432	0.353	6,637	470	F32	0.332	21,434	1,516
8341	0.357	3,005	216	G11	0.350	13,355	945
8331	0.355	3,383	242	G23	0.350	14,138	1,000
8332	0.355	3,196	229	I24	0.353	13,466	953
8327	0.354	3,665	263	I25	0.351	15,035	1,064
6423	0.355	3,369	241	11494	0.359	5,919	419
6454	0.350	3,395	243	11495	0.359	5,946	421
C01	0.343	12,904	913	12684	0.342	9,422	667
C12	0.344	12,876	911	13775	0.362	6,234	441
C20	0.350	12,634	894	13630	0.351	7,955	563
0C9	0.348	15,641	1,106	13628	0.358	6,788	480
2A5	0.371	7,075	501	13847	0.363	5,895	417
5A3	0.373	7,245	513	13848	0.363	5,881	416
1C2	0.355	13,152	930	KU0269	0.358	6,039	428
1C5	0.347	15,734	1,113	KU0278	0.358	6,079	430
2C2	0.350	14,757	1,044	KU0282	0.354	6,668	472

* These values give a conservative estimate of the uncertainty in because the relative uncertainty in each component of f was assumed to be 5% /2/.

Using the results from the calorimetric (P_{tot}) and the gamma scanning measurements, the correlation coefficient $C = P_{tot} / \frac{I}{f}$ in Equation 2-1 was determined by performing a least squares fit (Appendix B) of P_{tot} as a function of $\frac{I}{f}$. The values of C thus obtained are shown in Table 6-3. Because different detectors were used in January and June, two calibration curves had to be established for each of the following geometries: 8×8 (with fuel channel) and 8×8 (without fuel channel). All calibration curves can be found in Appendix B.

Table 6-3. The calibration constants for each assembly type determined from the calibration curves. Different detectors were used in the January and June measurement campaigns.

Assembly type	Fuel channel	Measurement campaign	Calibration constant (10^{-2} Ws)
8×8	Yes	January	3.23 ± 0.10
8×8	No	January	2.76 ± 0.04
		June	5.38 ± 0.05
KWU 15×15	No	January	3.10 ± 0.07
W15×15	No	January	3.25 ± 0.10
W 17×17	No	January	3.17 ± 0.05
SVEA 64	Yes	January	2.89 ± 0.09
SVEA100	Yes	January	2.90 ± 0.05
KWU9×9-5	Yes	January	3.23 ± 0.04

Using the calibration constant obtained for each fuel type, Equation 2-1 was used to predict the expected residual power for each assembly that was scanned. The residual power obtained from gamma scanning (P_{scan}) and the residual power obtained from the calorimetric measurements (P_{cal}) is shown in Tables 6-4 to 6-10 for each fuel type. In addition, Table D-1 in Appendix D gives a summary of the results for all the fuel types that were analyzed.

Table 6-4. Agreement between calorimetric measurements and gamma scanning for 8x8 assemblies with fuel channel.

Fuel type: 8x8 with fuel channel			
Fuel id	P _{cal} (W)	P _{scan} (W)	Difference (%)
1177	177.92	175.80	1.19
9329	222.82	223.80	-0.44
1186	140.78	139.72	0.75
1389	83.92	84.11	-0.22
1546	108.12	110.28	-1.99
1696	92.37	91.83	0.59
1704	83.98	83.22	0.90
2995	130.50	130.24	0.20
3838	126.80	119.45	5.79
6350	126.91	134.50	-5.98
10288	185.81	187.63	-0.98
12078	120.17	126.37	-5.16
14076	240.28	233.70	2.74
Relative Standard Deviation (%)	3.04		

Table 6-5. Agreement between calorimetric measurements and gamma scanning for 8x8 assemblies without fuel channels.

Fuel type: 8x8 without fuel channel			
Fuel id	P _{cal} (W)	P _{scan} (W)	Difference (%)
8338	169.50	168.45	0.62
8327	196.94	195.09	0.94
1136	95.23	95.52	-0.31
6423	174.21	178.27	-2.33
6432	184.44	183.04	0.76
8341	164.92	161.74	1.93
8331	187.03	182.05	2.66
8332	168.10	172.00	-2.32
8327	196.94	197.26	-0.16
6423	174.21	181.29	-4.07
6454	186.30	182.70	1.93
Relative Standard Deviation (%)	2.09		

Table 6-6. Agreement between calorimetric measurements and gamma scanning for W17×17 assemblies.

Fuel type: W17×17			
Fuel id	P _{cal} (W)	P _{scan} (W)	Difference (%)
0C9	491.17	496.14	-1.01
2A5	233.76	224.42	4.00
5A3	237.72	229.81	3.33
1C2	417.67	417.19	0.12
1C5	499.17	499.10	0.01
2C2	466.53	468.12	-0.34
3C1	470.23	471.75	-0.32
3C5	501.41	502.47	-0.21
3C9	468.42	469.18	-0.16
4C4	422.04	429.33	-1.73
4C7	498.75	505.26	-1.30
0E2	587.90	590.48	-0.44
0E6	487.75	482.23	1.13
1E5	468.77	461.09	1.64
Relative Standard Deviation (%)	1.66		

Table 6-7. Agreement between calorimetric measurements and gamma scanning for KWU15×15 assemblies.

Fuel type: KWU15×15			
Fuel id	P _{cal} (W)	P _{scan} (W)	Difference (%)
I09	507.94	509.56	-0.32
D27	456.05	458.94	-0.63
F25	396.75	389.96	1.71
D38	442.34	446.01	-0.83
E38	374.33	382.13	-2.08
E38	376.31	385.23	-2.37
E40	381.25	375.46	1.52
F14	381.81	378.68	0.82
F21	420.90	412.42	2.02
F32	691.99	658.88	4.78
G11	416.37	410.54	1.40
G23	420.63	434.62	-3.33
I24	410.07	413.94	-0.94
I25	445.79	462.19	-3.68
Relative Standard Deviation (%)	2.34		

Table 6-8. Agreement between calorimetric measurements and gamma scanning for SVEA 64 assemblies.

Fuel type: Svea 64			
Fuel id	P_{cal} (W)	P_{scan} (W)	Difference (%)
11494	166.04	171.05	-3.02
11495	167.55	171.82	-2.55
12684	282.74	272.28	3.70
13775	178.35	180.16	-1.01
Relative Standard Deviation (%)			
	3.07		

Table 6-9. Agreement between calorimetric measurements and gamma scanning for SVEA 100 assemblies.

Fuel type: Svea 100			
Fuel id	P_{cal} (W)	P_{scan} (W)	Difference (%)
13630	235.67	231.03	1.97
13628	194.04	197.12	-1.59
13847	169.58	171.20	-0.96
13848	170.70	170.79	-0.05
Relative Standard Deviation (%)			
	1.55		

Table 6-10. Agreement between calorimetric measurements and gamma scanning for KWU9x9-5 assemblies.

Fuel type: KWU9x9-5			
Fuel id	P_{cal} (W)	P_{scan} (W)	Difference (%)
KU0269	192.69	195.02	-1.21
KU0278	195.44	196.29	-0.44
KU0282	218.46	215.31	1.44
Relative Standard Deviation (%)			
	1.36		

7 Summary

Gamma scanning of 86 spent nuclear fuel assemblies was performed during two measurement campaigns. Of these, 64 assemblies were used in the analysis for the determination of the residual thermal power, while 22 assemblies were set aside from the analysis due to the reasons as accounted for in Section 4.1. The excluded assemblies will be included in further studies.

Typical measuring times were about 15 minutes and the total time spent on each assembly was about 30 minutes including fuel handling.

In the first step of the analysis, the calibration constants were calculated for each fuel type. In the second step, the residual power was determined. The values of the residual power obtained agreed with the values obtained from calorimetric measurements with a standard deviation of 2.23% (Table 13-1, Appendix D).

It has also been shown that the new data acquisition and analysis software is applicable for collecting gamma-scanning data for determining the residual thermal power. Furthermore, both software tools have been used to carry out the necessary computations including the regression fit for computing the calibration constants. With the developed software, it will be possible to obtain the value of the residual thermal power for a given assembly directly after scanning.

8 Outlook

The overall accuracy of 2.23% stated above refers to the whole set of fuel assemblies considered. Within the various types of assemblies the accuracy varies, however, from about 1.4% to about 3.1%. Here one should bear in mind that some subsets consist of very few assemblies which makes the determined accuracy somewhat questionable. It is therefore strongly suggested that future work includes more fuel assemblies in these subsets.

To improve the gamma scanning technique for determination of residual power, the following areas will be considered:

- i) Integration of the current software tools (analysis and data acquisition) to provide an integrated gamma-scanning tool for the determination of residual thermal power.
- ii) The use of gamma transport coefficients /14/ (as calculated with a technique used in tomographic computations) to relate the calibration constants (C) for different fuel designs to each other. This would make it possible to construct one calibration curve for all fuel types. By using the gamma transport coefficients, it may be possible to handle the following cases:
 - a) Enable the determination of the residual power for fuel designs that are too few to be used in a calibration curve.
 - b) Enable the modelling and determination of the residual thermal power for fuel assemblies that have been reconstructed through the replacement or removal of fuel rods as well as assemblies that still contains control rods
 - c) Use of one single reference fuel assembly for calibration purposes. This would, in principle, facilitate the process of relating previous measurement geometries to current and future geometries.

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- /14/ A Tomographic Measurement Technique for Irradiated Nuclear Fuel Assemblies, Staffan Jacobsson Svärd, Acta Universitates Upsaliensis, (Uppsala 2004).

Fuel assemblies included in the assay

Tables A-1 and A-2 show the fuel assemblies that were included in the assay. The data for the January campaign are shown in Table 10-1 while the data for the June campaign are shown in Table A-2. The fuel assemblies that were not part of the analysis are listed in Table A-3.

Table A-1. This table shows the fuel assemblies that were scanned in the January measurement campaign.

Fuel id	Geometry	Fuel channel	Enrichment (%)	Declared BU (GWd/tU)	Cooling time (days)
1177	8x8	Yes	2.65	36.242	6,382
9329	8x8	Yes	2.92	41.127	5,240
1186	8x8	Yes	2.65	30.498	6,381
1389	8x8	Yes	2.20	19.481	7,862
1546	8x8	Yes	2.20	24.47	7,103
1696	8x8	Yes	2.20	20.87	7,097
1704	8x8	Yes	2.20	19.437	7,493
2995	8x8	Yes	2.70	29.978	7,867
3838	8x8	Yes	2.09	25.669	3,854
6350	8x8	Yes	2.88	27.674	6,440
10288	8x8	Yes	2.95	35.218	5,239
12078	8x8	Yes	2.58	25.16	5,310
14076	8x8	Yes	3.15	40.01	3,862
8338	8x8	No	2.91	34.83	5,283
8327	8x8	No	2.90	37.851	4,185
6423	8x8	No	2.90	35.109	5,282
6432	8x8	No	2.89	36.861	5,282
1136	8x8	No	2.25	25.498	7,158
I09	KWU15x15	No	3.20	40.188	5,362
D27	KWU15x15	No	3.25	39.676	7,201
F25	KWU15x15	No	3.20	35.352	7,203
D38	KWU15x15	No	3.25	39.403	7,557
E38	KWU15x15	No	3.20	33.973	7,557
E38	KWU15x15	No	3.20	33.973	7,564
E40	KWU15x15	No	3.20	34.339	7,559
F14	KWU15x15	No	3.20	34.009	7,203
F21	KWU15x15	No	3.20	36.273	6,852
F32	KWU15x15	No	3.20	50.962	5,366
G11	KWU15x15	No	3.19	35.463	6,497
G23	KWU15x15	No	3.21	35.633	6,497
I24	KWU15x15	No	3.20	34.337	6,109
I25	KWU15x15	No	3.20	36.859	5,743
C01	W15x15	No	3.095	36.688	7,956
C12	W15x15	No	3.095	36.385	7,955

Fuel id	Geometry	Fuel channel	Enrichment (%)	Declared BU (GWd/tU)	Cooling time (days)
C20	W15x15	No	3.095	35.720	6,494
0C9	W17x17	No	3.10	38.442	6,074
2A5	W17x17	No	2.10	20.107	6,823
5A3	W17x17	No	2.10	19.699	6,822
1C2	W17x17	No	3.10	33.318	6,075
1C5	W17x17	No	3.10	38.484	6,074
2C2	W17x17	No	3.10	36.577	6,074
3C1	W17x17	No	3.10	36.572	6,074
3C5	W17x17	No	3.10	38.373	6,073
3C9	W17x17	No	3.10	36.56	6,074
4C4	W17x17	No	3.10	33.333	6,079
4C7	W17x17	No	3.10	38.37	6,074
0E2	W17x17	No	3.10	41.628	5,305
0E6	W17x17	No	3.10	35.993	5,306
1E5	W17x17	No	3.10	34.638	5,305
11494	SVEA 64	Yes	2.92	32,431	5,305
11495	SVEA 64	Yes	2.91	32,431	5,303
12684	SVEA 64	Yes	2.902	46,648	4,197
13775	SVEA 64	Yes	2.85	32,837	4,218
13630	SVEA 100	Yes	2.711	40,363	4,236
13628	SVEA 100	Yes	2.711	35,619	4,236
13847	SVEA 100	Yes	2.77	31,275	4,575
13848	SVEA 100	Yes	2.77	31,275	4,575
KU0269	KWU9x9-5	Yes	2.938	35,113	4,548
KU0278	KWU9x9-5	Yes	2.938	35,323	4,267
KU0282	KWU9x9-5	Yes	2.938	37,896	4,262

Table A-2. The fuel assemblies that were scanned in the June measurement campaign.

Fuel id	Geometry	Fuel channel	Enrichment (%)	Declared BU (GWd/tU)	Cooling time (days)
8341	8x8	No	2.89	34.099	5,415
8331	8x8	No	2.89	35.903	5,010
8332	8x8	No	2.90	34.977	5,415
8327	8x8	No	2.90	37.851	4,317
6423	8x8	No	2.90	35.109	5,415
6454	8x8	No	2.90	37.236	6,137

Table A-3. Assemblies that were not part of the analysis.

Fuel id	Status	Geometry
5829	Rebuilt	8x8
3058	Rebuilt	8x8
3064	Rebuilt	8x8
6478	Rebuilt	8x8
5F2	Rebuilt	AA17x17
2014	Rebuilt	8x8
2018	Rebuilt	8x8
2048	Rebuilt	8x8
2074	Rebuilt	8x8
2118	Rebuilt	8x8
3054	Rebuilt	8x8
0582	Rebuilt	8x8
0596	Rebuilt	8x8
0900	Rebuilt	8x8
3C4	Rebuilt	W17x17
I20	Rebuilt	KWU15x15
C42	Rebuilt	W15x15
A05	Contains control rods	W15x15
A11	Contains control rods	W15x15
5535	Too few	8x8
1377	Too few	8x8
KU0100	Too few	8x8-2

Calibration curves

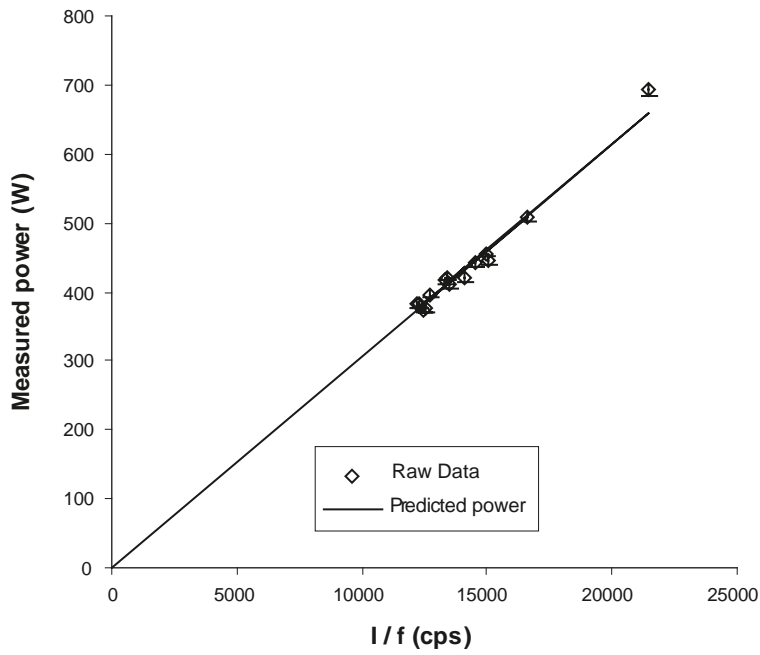


Figure B-1. The calibration curve for the residual power in KWU15×15 fuel assemblies. The estimated deviation of the least square fit is: 2.34%.

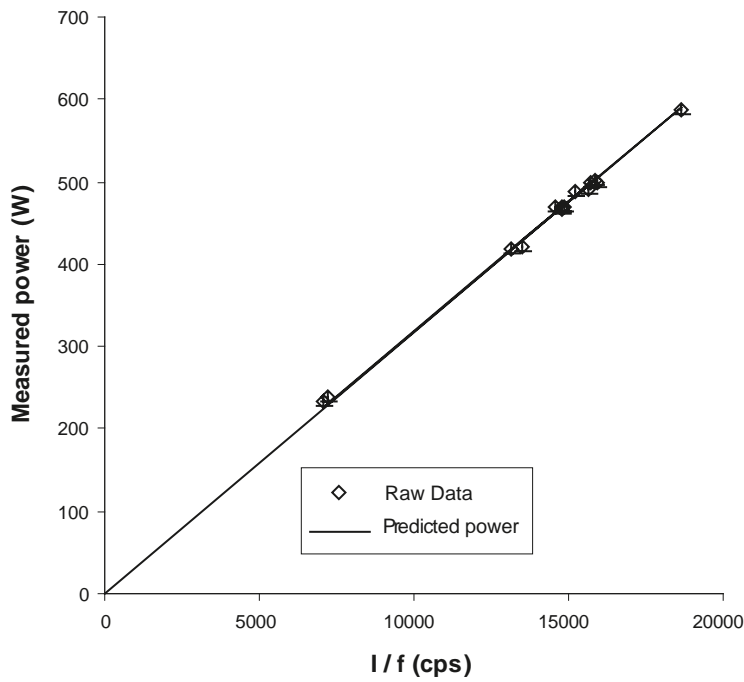


Figure B-2. The calibration curve for the residual power in W17×17 fuel assemblies. The estimated deviation of the least square fit is: 1.66%.

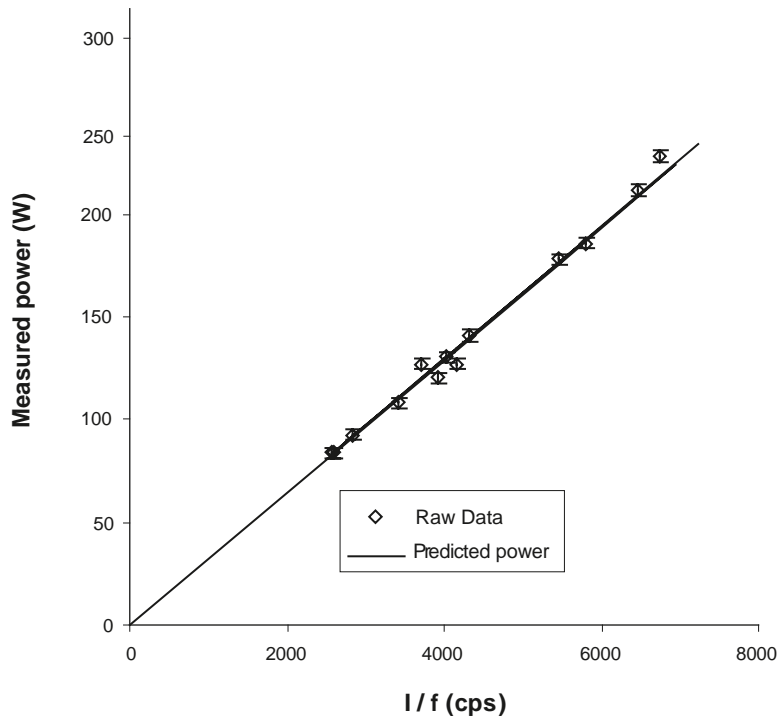


Figure B-3. The calibration curve for the residual power of 8×8 assemblies with fuel channel. These assemblies were scanned in the January campaign. The estimated deviation of the least square fit is: 3.15%.

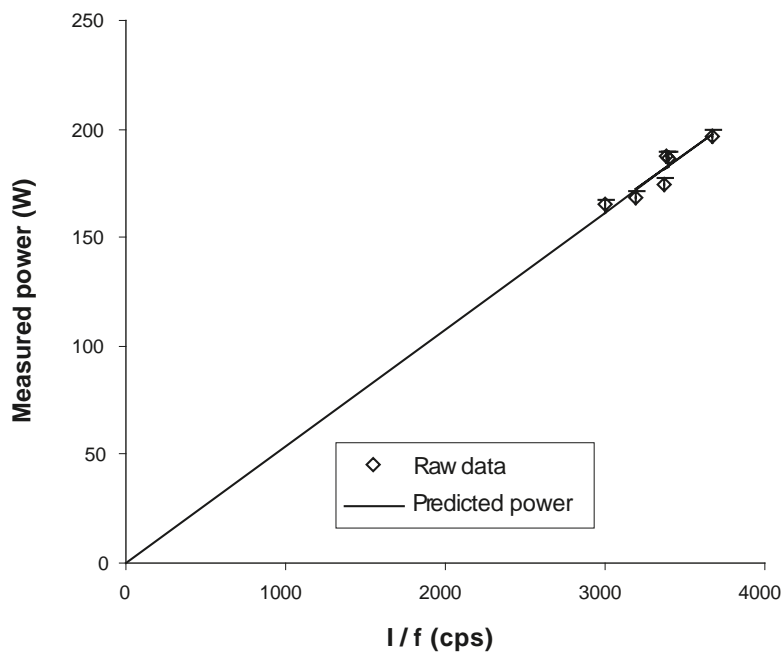


Figure B-4. The calibration curve for the residual power in 8×8 assemblies without fuel channels. These assemblies were scanned in the June campaign. The estimated deviation of the least square fit is: 2.70%.

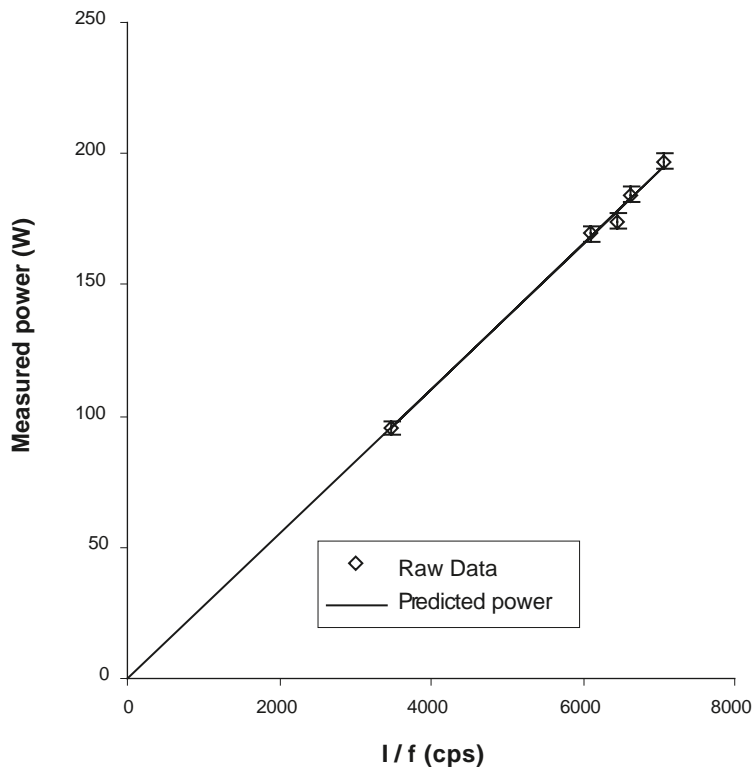


Figure B-5. The calibration curves for the residual power of 8×8 assemblies without fuel channel. These assemblies were scanned in the January campaign. The estimated deviation of the least square fit is: 1.35%.

Results from the calorimetric measurements

Table C-1. Results from the calorimetric measurements of the thermal residual power.

Fuel id	Calorimetric power (W)	Escape power (W)	Total power (W)	Uncertainty (W)	Fuel id	Calorimetric power (W)	Escape power (W)	Total power (W)	Uncertainty (W)
1177	173.38	4.54	177.92	2.82	6454	181.27	5.03	186.30	2.86
9329	217.40	5.42	222.82	3.05	C01	406.3	9.45	415.75	5.14
1186	137.48	3.3	140.78	2.65	C12	401.64	8.7	410.34	5.11
1389	81.74	2.18	83.92	2.41	C20	416.39	9.64	426.03	5.19
1546	105.39	2.73	108.12	2.50	C42	431.7	10.58	442.28	5.27
1696	90.04	2.33	92.37	2.44	0C9	478.62	12.55	491.17	5.53
1704	81.84	2.14	83.98	2.41	2A5	227.92	5.84	233.76	4.34
2014	80.80	1.9	82.70	2.40	5A3	231.89	5.83	237.72	4.36
2018	82.32	1.99	84.31	2.41	1C2	407.29	10.38	417.67	5.15
2048	92.36	2.23	94.59	2.45	1C5	486.51	12.66	499.17	5.57
2995	127.29	3.21	130.50	2.60	2C2	454.81	11.72	466.53	5.40
3838	122.56	3.38	125.94	2.58	3C1	458.29	11.94	470.23	5.42
6350	123.69	3.22	126.91	2.58	3C4	484.91	12.43	497.34	5.56
10288	181.18	4.63	185.81	2.86	3C5	488.88	12.53	501.41	5.58
12078	116.91	3.26	120.17	2.56	3C9	456.47	11.95	468.42	5.41
14076	233.70	6.58	240.28	3.15	4C4	411.41	10.63	422.04	5.17
2074	95.47	2.3	97.77	2.46	4C7	485.91	12.84	498.75	5.57
2118	95.92	2.38	98.30	2.46	0E2	572.38	15.52	587.90	6.06
3054	137.37	3.67	141.04	2.65	0E6	475.12	12.63	487.75	5.51
5535	82.40	2.2	84.60	2.41	1E5	456.86	11.91	468.77	5.41
1377	54.80	1.43	56.23	2.31	I09	494.53	13.41	507.94	5.62
8338	164.86	4.64	169.50	2.78	D27	444.65	11.4	456.05	5.34
8327	190.97	5.97	196.94	2.92	F25	386.71	10.04	396.75	5.05
582	89.35	2.3	91.65	2.44	D38	431.59	10.75	442.34	5.27
596	85.33	2.96	88.29	2.43	E38	366.95	9.36	376.31	4.95
710	89.94	3.01	92.95	2.44	E38	366.95	9.36	376.31	4.95
900	93.68	2.8	96.48	2.46	E40	372.24	9.01	381.25	4.97
1136	93.15	2.63	95.78	2.45	F14	372.47	9.34	381.81	4.98
6423	169.05	5.16	174.21	2.80	F21	410.2	10.7	420.90	5.17
6432	178.90	5.54	184.44	2.86	F32	675.4	16.59	691.99	6.66
8341	158.06	4.79	162.85	2.75	G11	405.75	10.62	416.37	5.14
8331	181.30	5.73	187.03	2.87	G23	409.56	11.07	420.63	5.17
8332	162.95	5.15	168.10	2.77	I20	393.78	9.68	403.46	5.08
8327	190.97	5.97	196.94	2.92	I24	398.24	11.83	410.07	5.11
6423	169.05	5.16	174.21	2.80	I25	433.99	11.8	445.79	5.29
11494	161.66	4.38	166.04	2.77	13847	164.66	4.92	169.58	2.78
11495	163.26	4.29	167.55	2.77	13848	165.64	5.06	170.70	2.79
12684	274.91	7.83	282.74	3.39	KU0269	187.13	5.56	192.69	2.90
13775	173.07	5.28	178.35	2.83	KU0278	189.90	5.54	195.44	2.91
13630	228.58	7.09	235.67	3.12	KU0282	212.54	5.92	218.46	3.03
13628	188.24	5.8	194.04	2.90					

Summary of the measurement results

Table D-1. Results of the measurements for all fuel types that were used in the analysis.

Fuel id	Pcal (W)	Pscan (W)	Difference (%)	Fuel id	Pcal (W)	Pscan (W)	Difference (%)
1177	177.92	175.80	1.19	3C5	501.41	502.47	-0.21
9329	222.82	223.80	-0.44	3C9	468.42	469.18	-0.16
1186	140.78	139.72	0.75	4C4	422.04	429.33	-1.73
1389	83.92	84.11	-0.22	4C7	498.75	505.26	-1.30
1546	108.12	110.28	-1.99	0E2	587.9	590.48	-0.44
1696	92.37	91.83	0.59	0E5	487.75	482.23	1.13
1704	83.98	83.22	0.90	1E5	468.77	461.09	1.64
2995	130.50	130.24	0.20	l09	507.94	509.56	-0.32
3838	126.80	119.45	5.79	D27	456.05	458.94	-0.63
6350	126.91	134.50	-5.98	F25	396.75	389.96	1.71
10288	185.81	187.63	-0.98	D38	442.34	446.01	-0.83
12078	120.17	126.37	-5.16	E38	374.33	382.13	-2.08
14076	240.28	233.70	2.74	E38	376.31	385.23	-2.37
8338	169.50	168.45	0.62	E40	381.25	375.46	1.52
8327	196.94	195.09	0.94	F14	381.81	378.68	0.82
1136	95.23	95.52	-0.31	F21	420.9	412.42	2.02
6423	174.21	178.27	-2.33	F32	691.99	658.88	4.78
6432	184.44	183.04	0.76	G11	416.37	410.54	1.40
8341	164.92	161.74	1.93	G23	420.63	434.62	-3.33
8331	187.03	182.05	2.66	l24	410.07	413.94	-0.94
8332	168.10	172.00	-2.32	l25	445.79	462.19	-3.68
8327	196.94	197.26	-0.16	11494	166.04	171.05	-3.02
6423	174.21	181.29	-4.07	11495	167.55	171.82	-2.55
6454	186.30	182.70	1.93	12684	282.74	272.28	3.70
C01	415.75	420.40	-1.12	13775	178.35	180.16	-1.01
C12	410.34	419.48	-2.23	13630	235.67	231.03	1.97
C20	426.03	411.58	3.39	13628	194.04	197.12	-1.59
0C9	491.17	496.14	-1.01	13847	169.58	171.20	-0.96
2A5	233.76	224.42	4.00	13848	170.7	170.79	-0.05
5A3	237.72	229.81	3.33	KU0269	192.69	195.02	-1.21
1C2	417.67	417.19	0.12	KU0278	195.44	196.29	-0.44
1C5	499.17	499.10	0.01	KU0282	218.46	215.31	1.44
2C2	466.53	468.12	-0.34				
3C1	470.23	471.75	-0.32				
Relative Standard Deviation (%)	2.23						

Clab – Calibration curve for calorimetric measurement of fuel assemblies

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Contents

1	Background	101
2	Measurement method	103
2.1	Temperature increase method	103
2.2	Circulation method	103
3	Equipment	105
4	Measurement uncertainty	107
5	Calibration measurements	109
5.1	Temperature increase method	109
5.1.1	Calibration according to the temperature increase method	109
5.1.2	Determination of the decay heat power of a fuel assembly with the calibration curve	113
5.1.3	Error analysis of temperature increase method	115
5.1.4	Sensitivity analysis for errors in data for fuel assemblies	115
5.2	Circulation method	117
5.2.1	Calibration according to the circulation method	117
5.2.2	Calculation of a fuel assembly's decay heat power by means of circulation calibration	118
5.3	Heat losses due to gamma radiation	120
6	References	123
Appendix 1	Illustration of calorimeter system 251	125
Appendix 2	Primary data temperature increase method	127
Appendix 3	Primary data circulation method	129
Appendix 4	Example of calculation of KA in connection with calibration according to circulation method	131
Appendix 5	F factors for calculation of gamma escape power	133

1 Background

In order to optimize the final repository for spent fuel, it is desirable to be able to measure the decay heat power in the fuel. Decay heat power measurement is also needed to verify programs that calculate the decay heat power. This report describes how decay heat power measurement has been carried out in Clab with a specially designed calorimeter (system 251) according to the temperature increase method and the circulation method.

An illustration of the calorimeter is shown in Appendix 1 and a detailed description is found in reference 1.

2 Measurement method

2.1 Temperature increase method

The principle of calorimetric measurement of decay heat power according to the temperature increase method is that the fuel assembly is placed in a limited and insulated water volume, in this case a calorimeter. Depending on how much decay heat power the fuel emits, the temperature of the water in the calorimeter will increase at different rates. The rate of temperature increase will be a direct measure of the decay heat power of the fuel. In order to permit the rate of temperature increase to be related to a decay heat power value, a calibration curve has been plotted.

2.2 Circulation method

Just as in the case of the temperature increase method, the fuel assembly is placed in the calorimeter. In contrast to the temperature increase method, a known flow is started to and from the otherwise insulated calorimeter. After a while a steady state will be reached where the temperature in the calorimeter is stable. In this state the decay heat power and the rate of heat emission to the surrounding water are equal. Most of the heat is carried away with the flow pumped through the calorimeter. Furthermore, the temperature difference between the calorimeter water and the surrounding water results in a convective heat loss that passes through the insulated wall of the calorimeter. The heat lost to the water circulating through the calorimeter can be determined with good precision by measuring the mass flow and the temperature of the water both as it enters and as it leaves the calorimeter. But in order to take into account the heat loss through the calorimeter wall, calibration measurements have been performed.

3 Equipment

Clab system 251 (calorimeter system for decay heat power measurement) was used for the measurements.

The calorimeter system consists of:

- A console (8.PAM.406) containing a control computer and measurement instruments.
- A connection module with valves, calibration tank and pump.
- A calorimeter where the fuel assembly and the electric heater (BWR dummy) are placed.

The fuel assembly whose decay heat power is to be measured is placed in the calorimeter, whose lid keeps the calorimeter watertight. An electric heater with the same geometry as a real BWR assembly is used for calibration. The heater has 64 “fuel rods” with 261 m of heating cable with a resistance of 0.1 ohm/m. From the console there is a 20 m one-way transport cable with a resistance of 0.012 ohm/m. Some of the heat will be lost in the transport cable. In order to produce the heat power that is delivered by the electric heater, the power value must be multiplied by $261 \times 0.1 / (261 \times 0.1 + 2 \times 20 \times 0.012) = 0.982$. For practical reasons the electric heater has its own lid to ensure watertightness. There are temperature sensors in the calorimeter whose values are recorded in the control computer. All activations of valves and pumps take place from the control computer program’s process display.

4 Measurement uncertainty

The system description for system 251 (ref /1/) gives the error limits for the equipment:

Temperature	$\pm 0.01^{\circ}\text{C}$
Mass flow	$\pm 0.5\%$ within 0.36 kg/min–7.2 kg/min
Power output	$\pm 0.25\%$ within 50 W–1,000 W
Gamma dose rate	$\pm 5\%$ within 0–100 Gy/h
Mass	$\pm 0.02\%$

These errors contribute to the random errors in the measurements. A detailed analysis of measurement uncertainty is found in Section 3, part 4 in this report.

5 Calibration measurements

Calibration measurements were performed in the range 50 to 1,000 Watts. A total of 49 calibration measurements were performed according to the temperature increase method 26 according to the circulation method. Following is a description of how the calibrations were carried out and their results.

5.1 Temperature increase method

5.1.1 Calibration according to the temperature increase method

- The fuel-like heating element is placed in the calorimeter.
- The electric heater is set to the power output to be run.
- Deionized water: 733 water is connected to the calorimeter to cool it to 1.5°C below the temperature of the surrounding pool water.
- All measurement data are logged until the temperature in the calorimeter is 1.5°C warmer than the surrounding water.
- The temperature difference between the calorimeter and the surrounding pool water (KA502–KA531, see Table 5-1 below) is plotted as a function of time in a graph (see Figure 5-1). Time zero is when the temperature difference between the calorimeter and the pool (DT) is zero. A second degree curve is fitted as closely as possible to the plotted points.
- The slope of the second degree curve in the point $DT = 0^{\circ}\text{C}$ is determined (1.35382×10^{-4} in Figure 5-1).
- The calibration curve is drawn by plotting the power values (KA901, see Table 5-1 below) for the different measurements as a function of the slope in the point $DT = 0^{\circ}\text{C}$. The power value is multiplied by 0.982 as described in Chapter 3. This run thus contributes the point (1.35382×10^{-4} ; $149.9 \times 0.982\text{W}$) to the calibration curve (Figure 5-2).

Table 5-1 shows an extract from the primary data saved in connection with a calibration measurement according to the temperature increase method. More complete primary data are found in Appendix 2.

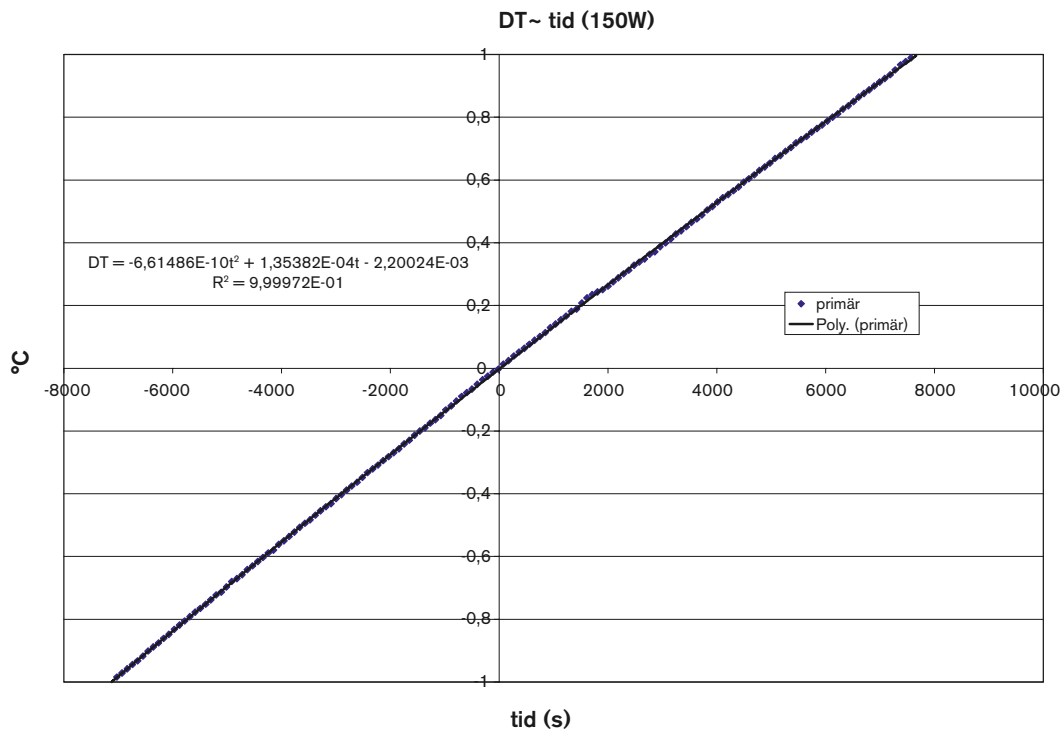


Figure 5-1. Temperature difference between calorimeter and surrounding pool as a function of time. Measurement with starting date 26 May 2003 and with a box on the electric heater.

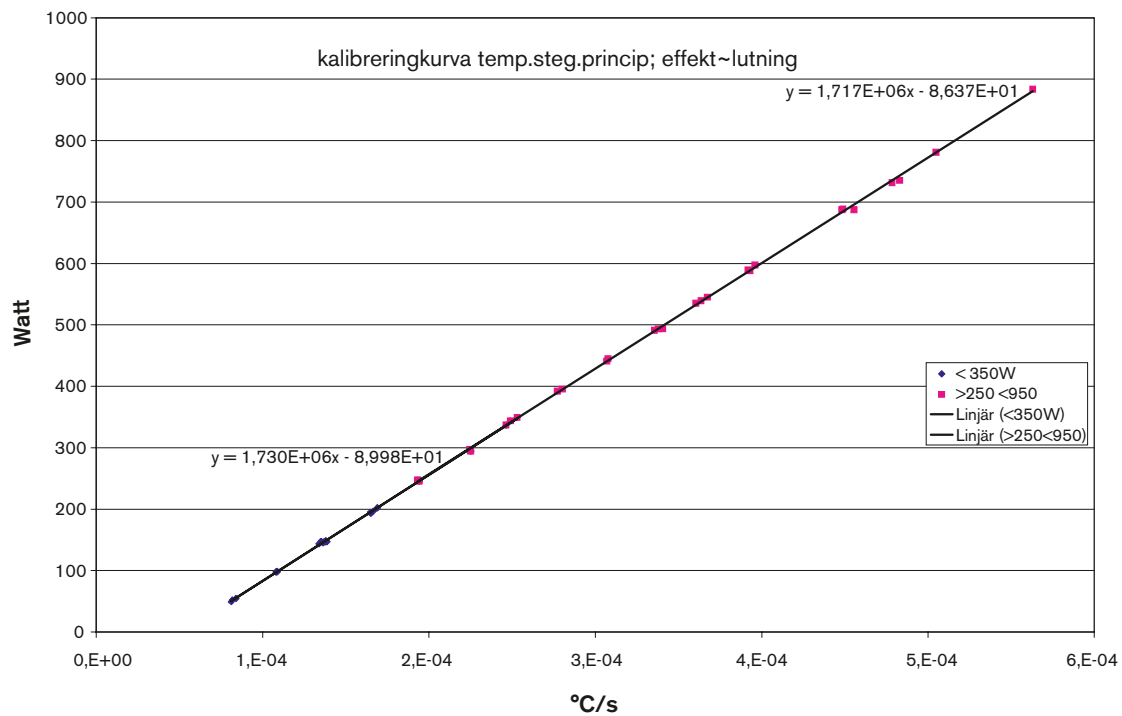


Figure 5-2. Calibration curve for temperature increase method. Power as a function of slope.

Table 5-1. Primary data in calibration according to the temperature increase method.

Tinfo1		Tinfo2						
Fredrik Sturek								
Kalib. Temperaturstegringsprincipen		150Wtempstegbox030526						
KA701	KA702							
360	16							
0.00424	0.00421							
KA703	KA704							
76	-514							
0.00449	0.00425							
KA705	KALIB.TEMP							
-132	21.5							
0.00451	Förstärkning							
	023							
Mon May 26 2003 08:30:22								
Logg_tid	T1	T5	T6	T16	T17	T18	T20	T21
tid	_251KA901_	_251KA502_	_251KA502_	_251KA751_	_251KA101	_251KA521_	_251KA531_	_251KA531_
	A_kW	B1	B2	Vikt		B1	B1	B2
sekunder	W	°C	°C	kg	bar	°C	°C	°C
60	149.52							
96		18.276	18.279				19.743	19.749
120	149.81							
180	149.73							
192		18.268	18.273			18.218	19.752	19.751
240	150.10							
288		18.271	18.274				19.749	19.746
300	150.16			11.17				
360	150.16							
384		18.269	18.275			18.248	19.745	19.751
420	149.73							
480	149.90	18.291	18.297				19.748	19.747
540	149.69							
576		18.307	18.313			18.280	19.746	19.748
600	149.78			11.17	1.39			
660	149.24							
672		18.321	18.325				19.746	19.748
720	149.49							
768		18.337	18.338			18.310	19.746	19.748
780	149.95							
840	150.41							
864		18.351	18.355				19.739	19.749
900	150.13			11.17				
960	150.32	18.364	18.368			18.342	19.744	19.747
1020	149.89							
1056		18.379	18.383				19.746	19.742
1080	150.41							
1140	150.36							
1152		18.391	18.397			18.368	19.746	19.747
1200	150.30			11.17	1.37			
1248		18.404	18.411				19.746	19.749
1260	150.36							
1320	150.93							
1344		18.422	18.425			18.397	19.746	19.748
1380	150.51							
1440	149.54	18.436	18.439				19.746	19.746
1500	149.49			11.17				
1536		18.449	18.453			18.425	19.745	19.749
1560	149.42							
1620	149.84							
1632		18.465	18.466				19.747	19.750
1680	149.87							
1728		18.478	18.483			18.455	19.747	19.749
1740	149.74							
1800	149.57			11.18	1.38			

Table 5-2 below shows the values on which the calibration curve in Figure 5-2 is based.

Table 5-2. Power and slope values included in the calibration curve (Figure 5-2).

Starting date	Power (W)	Pool temp (°C)	Slope at $\Delta T = 0^\circ\text{C}$
030508box	49	19.24	8.11E-05
30408	52	18.53	8.18E-05
30210	55	18.33	8.40E-05
30314	97	18.02	1.08E-04
030512box	98	19.64	1.09E-04
30327	98	18.40	1.08E-04
030528box	98	19.76	1.08E-04
30428	99	19.00	1.09E-04
040202box?	143	19.89	1.34E-04
21216	145	18.80	1.36E-04
40203box	146	19.95	1.37E-04
031103box	147	19.07	1.38E-04
30321	147	18.24	1.37E-04
30526box	147	19.75	1.35E-04
30325	147	18.27	1.35E-04
30107	147	19.12	1.38E-04
30430box	147	19.12	1.39E-04
30407	149	18.51	1.38E-04
030516box	193	19.60	1.65E-04
30422	195	18.91	1.65E-04
30220	197	17.93	1.67E-04
040205box	202	19.91	1.69E-04
040204box	202	19.94	1.69E-04
30411	245	18.59	1.94E-04
30319	246	18.15	1.94E-04
030520box	247	19.83	1.93E-04
30502box	295	19.11	2.25E-04
30131	297	18.63	2.25E-04
30324	337	18.23	2.46E-04
31104box	343	19.10	2.49E-04
030521box	349	19.84	2.53E-04
030519box	392	19.22	2.77E-04
30320	395	18.23	2.80E-04
30114	441	19.05	3.07E-04
030522box	445	19.85	3.08E-04
30321	491	18.22	3.36E-04
30505box	493	19.10	3.41E-04
030601box	493	19.69	3.38E-04
030523box	535	19.84	3.61E-04
030524box	539	19.81	3.64E-04
30327	545	18.39	3.67E-04

Starting date	Power (W)	Pool temp (°C)	Slope at $\Delta T = 0^\circ\text{C}$
30423	588	18.91	3.93E-04
30326	589	18.34	3.92E-04
30124	597	18.80	3.96E-04
30325	687	18.27	4.48E-04
30318	687	18.07	4.56E-04
30424	689	18.91	4.49E-04
30328	731	18.41	4.78E-04
30506box	735	19.10	4.83E-04
30317	781	18.01	5.05E-04
30324	884	18.22	5.63E-04

The decay heat power of the BWR assemblies to be measured is expected to lie between 60 and 350 Watts, so a linear relationship was sought for the points between 50 and 350 Watts. The relationship is: $\text{Power} = 1.73 \times 10^6 \times \text{slope} - 90.0$. The standard deviation for the expression is ± 1.9 Watts. The decay heat power of the PWR assemblies to be measured is expected to lie between 260 and 920 Watts, so a linear relationship was sought for the points between 250 and 950 Watts. The relationship is: $\text{Power} = 1.72 \times 10^6 \times \text{slope} - 86.4$. The standard deviation for the expression is ± 3.4 Watts.

5.1.2 Determination of the decay heat power of a fuel assembly with the calibration curve

- The fuel assembly is placed in the calorimeter.
- Deionized water: 733 water is connected to the calorimeter to cool it to 1.5°C below the temperature of the surrounding pool water.
- All measurement data are logged until the temperature in the cylinder is 1.5°C warmer than the surrounding water.
- The temperature difference between the cylinder and the surrounding pool water (KA502–KA531) is plotted as a function of time in a graph in the same way as in calibration. Time zero is the time when the temperature difference between the calorimeter and the pool (DT) is zero. A second degree curve is fitted as closely as possible to the plotted points.
- The slope of the second degree curve in the point $\text{DT} = 0^\circ\text{C}$ is determined.

The fuel assembly and the electric heater do not have the same volume, which means that the amount of energy needed to heat everything one degree differs slightly. The amount of energy per degree ($\text{kJ}/^\circ\text{C}$) is calculated in Table 5-3. Multiplying the slope by the ratio (at the bottom of Table 5-3) between the fuel assembly in question and the heater compensates for the difference between heater and fuel assembly. The corrected slope value is then used to determine the decay heat power of the fuel assembly by using the functions obtained in the calibration curve in Figure 5-2.

Table 5-3. Compensation for volume difference between electric heater and fuel assembly.

		Electric heater (fuel dummy)		Fuels		
		without box	with box	6432	9329	5A3
Volumes	Vol (dm ³)					
Calorimeter with truss and BWR guide	389.80	389.80	389.80	389.80	389.80	
Calorimeter with truss						390.20
Truss	5.25					
BWR guide	0.40					
Spacer	1.41			1.41		
Electric heater	13.30	13.30	13.30			
Plates for electric heater	2.27		2.27			
Guide	0.51	0.51	0.51			
Boxed BWR	36.2				36.2	
Unboxed BWR	29.0			29.0		
Element	76.7					76.72
Free water (dm³)		375.99	373.72	359.38	353.62	313.48
Quantity of steel in calorimeter						
Shell (kg)		112.33	112.33	112.33	112.33	112.33
Bottom (kg)		2.15	2.15	2.15	2.15	2.15
Nozzle (kg)		48.36	48.36	48.36	48.36	48.36
Lid (kg)				3.42	3.42	3.42
Circulation line (kg)		5.84	5.84	5.84	5.84	5.84
Truss (kg)		42.03	42.03	42.03	42.03	42.03
BWR guide (kg)		3.23	3.23	3.23	3.23	
Spacer (kg)				10.90		
Quantity of steel in heater (kg)						
Electric heater (kg)		71.70	71.70			
Heater lid (kg)		5.38	5.38			
Guide (kg)		4.11	4.11			
Plates (kg)			18.16			
Total quantity of steel (kg)		295.13	313.29	228.26	217.36	214.13
Quantity of steel in calorimeter (kg)		295.13	313.29	228.26	217.36	214.13
Quantity of water in calorimeter (kg)		375.99	373.72	359.38	353.62	313.48
Quantity of Cu in heater (kg)		3.48	3.48			
Quantity of plastic in heater (kg)		8.61	8.61			
Quantity of Zr in fuel (kg)				48.0	84.5	117.0
Quantity of steel/inc in fuel (kg)				4.2	9.2	20.90
Quantity of UO ₂ in fuel (kg)				201.5	202.9	523.80
Cp						
MCp for steel in calorimeter (kJ/°C)	0.46	135.76	144.11	105.00	99.99	98.50
MCp for water in calorimeter (kJ/°C)	4.18	1,572.18	1,562.69	1,502.74	1,478.65	1,310.82
MCp for Cu in heater (kJ/°C)	0.39	1.34	1.34			
MCp for plastic in heater (kJ/°C)	1.00	8.61	8.61			
MCp for Zr in fuel (kJ/°C)	0.28			13.43	23.67	32.76
MCp for steel/inc in fuel (kJ/°C)	0.46			1.93	4.24	9.61
MCp for UO ₂ in fuel (kJ/°C)	0.23			46.34	46.67	120.47
Total MCp (kJ/°C)		1,717.9	1,716.7	1,669.4	1,653.2	1,572.17
K			0.999	0.972	0.962	0.915

Table 5-4 shows the results of the decay heat power determination of 3 fuel assemblies in the summer of 2003.

Table 5-4. Fuel measurements performed in the summer of 2003.

Fuel	Date	Pool temp °C	Power PA2 Watt	Slope °C/S	Volume-corr slope	Fuel output Watts
9329	30602	19.68	127.7	1.85E-04	1.78E-04	217 ± 4
9329	30604	19.68	128.1	1.86E-04	1.79E-04	219 ± 4
6432	30610	19.77	127.8	1.61E-04	1.56E-04	180 ± 4
6432	30612	19.79	128.1	1.63E-04	1.58E-04	184 ± 4
5A3	30613	19.76	128.5	2.03E-04	1.85E-04	232 ± 7
5A3	30616	19.66	128.1	2.02E-04	1.85E-04	231 ± 7
5A3	30618	19.74	127.8	2.06E-04	1.89E-04	238 ± 7

5.1.3 Error analysis of temperature increase method

In connection with the calibration, a calibration curve was plotted (Figure 5-2) from the calibration values presented in Table 5-2. A regression analysis gave a standard deviation of ± 1.9 Watts for the curve that is planned to be used for BWR assemblies and ± 3.4 Watts for the curve that is planned to be used for PWR assemblies.

5.1.4 Sensitivity analysis for errors in data for fuel assemblies

For certain fuel assemblies there will be some uncertainty in the volume data. Table 5-5 below shows what it means if the (unboxed) volume of BWR assembly 6432 is changed from 29 litres to 26 litres, and if the volume of the PWR assembly is changed from 76.7 litres to 69 litres. The volume compensation parameter is changed from 0.972 to 0.979 and for the BWR assembly and from 0.915 to 0.934 for the PWR assembly. A 10% change in the volume of the BWR assembly entails a 0.7% change in the volume compensation parameter. In the case of the PWR assembly, a 10% change in volume entails a 2.1% change in the volume compensation parameter. The change in the calculated output is as great as the change in the volume compensation parameter.

Table 5-5. Compensation for volume difference between electric heater and fuel assembly. Changed fuel volumes in comparison with Table 5-3.

	Vol (dm ³)	Electric heater (fuel dummy)		Fuels		
		without box	with box	6432	9329	5A3
Volumes						
Calorimeter with truss and BWR guide	389.80	389.80	389.80	389.80	389.80	
Calorimeter with truss						390.2
Truss	5.25					
BWR guide	0.40					
Spacer	1.41			1.41		
Electric heater	13.30	13.30	13.30			
Plates for electric heater	2.27		2.27			
Guide	0.51	0.51	0.51			
Boxed BWR	36.2				36.2	
Unboxed BWR	26.0			26.0		
Element	69.0					69.0
Free water (dm³)		375.99	373.72	362.39	353.62	321.2
Quantity of steel in calorimeter						
Shell (kg)		112.33	112.33	112.33	112.33	112.33
Bottom (kg)		2.15	2.15	2.15	2.15	2.15
Nozzle (kg)		48.36	48.36	48.36	48.36	48.36
Lid (kg)				3.42	3.42	3.42
Circulation line (kg)		5.84	5.84	5.84	5.84	5.84
Truss (kg)		42.03	42.03	42.03	42.03	42.03
BWR guide (kg)		3.23	3.23	3.23	3.23	
Spacer (kg)				10.90		
Quantity of steel in heater (kg)						
Electric heater (kg)		71.70	71.70			
Heater lid (kg)		5.38	5.38			
Guide (kg)		4.11	4.11			
Plates (kg)			18.16			
Total quantity of steel (kg)		295.13	313.29	228.26	217.36	214.13
Quantity of steel in calorimeter (kg)		295.13	313.29	228.26	217.36	214.13
Quantity of water in calorimeter (kg)		375.99	373.72	362.39	353.62	321.20
Quantity of Cu in heater (kg)		3.48	3.48			
Quantity of plastic in heater (kg)		8.61	8.61			
Quantity of Zr in fuel (kg)				48.0	84.5	117.0
Quantity of steel/inc in fuel (kg)				4.2	9.2	20.90
Quantity of UO ₂ in fuel (kg)				201.5	202.9	523.80
	Cp					
MCp for steel in calorimeter (kJ/°C)	0.46	135.76	144.11	105.00	99.99	98.50
MCp for water in calorimeter (kJ/°C)	4.18	1,572.18	1,562.69	1,515.32	1,478.65	1,343.08
MCp for Cu in heater (kJ/°C)	0.39	1.34	1.34			
MCp for plastic in heater (kJ/°C)	1.00	8.61	8.61			
MCp for Zr in fuel (kJ/°C)	0.28			13.43	23.67	32.76
MCp for steel/inc in fuel (kJ/°C)	0.46			1.93	4.24	9.61
MCp for UO ₂ in fuel (kJ/°C)	0.23			46.34	46.67	120.47
Total MCp (kJ/°C)		1,717.9	1,716.7	1,682.02	1,653.2	1,604.43
K			0.999	0.979	0.962	0.934

5.2 Circulation method

5.2.1 Calibration according to the circulation method

- The fuel-like heating element is placed in the calorimeter.
- The electric heater is set to the power output to be run. The power value is multiplied by 0.982 as described in Chapter 3.
- A flow through the calorimeter is created where surrounding pool water is pumped into the calorimeter.
- The electric heater brings the hot water in the calorimeter to a state of equilibrium with respect to the temperature of the pumped-in water and the pumped-out water.
- All measurement data are logged at this state of equilibrium. An example of logged primary data is found in Appendix 3.
- If the temperature increases or decreases during measurement, the amount of energy absorbed/emitted by the water, the fuel and the steel in the calorimeter is calculated. The above parameter is not included explicitly in Table 5-6, but is included in Appendix 4.
- The heat power that is carried away with the flow (Flow power in Table 5-6 below) through the calorimeter is calculated by $E = F \times cp \times \Delta T$, where E is the power (Watts), F is the flow (kg/s), cp is the heat capacity of water J/(kg×s) and ΔT is the difference between the temperature of the incoming water (KA501) and the temperature of the outgoing water (KA502).
 - The water in the calorimeter is stirred with a circulation pump (PA2). This pump will add energy to the water in the calorimeter. The total power input to the pump is measured (PA2 power in Table 5-6), but it is not possible to determine how much of the pump power heats up the water in the calorimeter. Since the pump has the same operating case whether it is being used for calibration or measurement of fuel assemblies, the pump's contribution is cancelled out down to a few Watts of power variation. An attempt is made to take these small variations into account by adding the entire measured pump power to both the calibration and the fuel measurement.
- The electric heater's power output and the pump power are power supplied to the calorimeter. The flow power and the heat that is lost through the walls of the calorimeter are power removed from the calorimeter. Supplied and removed power must be equal, which makes it possible to calculate the power lost through the calorimeter wall E_k . Now the calculated E_k will always be greater than the actual value, since all pump power does not go into heating the water, but the difference can be assumed to be constant, which means that E_k will always be a set number of Watts greater than the actual value.
- The mean temperature difference ΔT_k is calculated by taking the difference between the temperature in the calorimeter (KA511) and the temperature of the surrounding water (KA531).
- A heat transfer coefficient is calculated by dividing E_k by ΔT_k : $KA = E_k / \Delta T_k$.

The calculations of KA that have been done can be seen in Table 5-6. Notice that the heater power is compensated as described in Chapter 3.

Table 5-6. Calculation of KA in calibration of the circulation method.

Name	Power heater Watts	DT °C	Tpool (KA531) °C	Tmc (KA511) °C	F kg/min	Flow power Watts	PA2 power Watts	Leakage Watts	KA W/°C	Difference^2
150Wcirk030329	145	2.02	18.5	20.6	1.46	205	128	68	32	8.86
380Wcirk030331	373	1.96	18.4	20.5	3.18	434	127	66	32	7.20
380Wcirk030401	373	1.95	18.4	20.5	3.18	432	126	68	33	2.27
840Wcirk030402	825	1.99	18.5	20.6	6.35	880	126	71	32	5.01
50W1cirk030403	57	1.78	18.5	20.4	0.97	121	127	64	32	4.15
150Wcirk030407	149	1.83	18.5	20.4	1.66	212	127	64	34	0.21
50Wcirk030409	52	1.79	18.6	20.4	0.95	118	127	61	33	1.37
300Wcirk030414	293	1.86	18.7	20.6	2.71	351	126	73	39	20.99
450Wcirk030417	442	2.17	18.8	21.1	3.28	496	126	72	32	6.34
200Wcirk030422	195	1.84	18.9	20.9	1.98	254	127	68	34	0.46
600Wcirk030423	588	1.87	18.9	21.1	4.89	639	126	75	34	0.13
700Wcirk030424	689	1.89	18.9	21.2	5.60	736	125	78	34	0.51
1000Wcirk030425	973	1.92	18.9	21.4	7.64	1,020	124	77	32	8.24
100Wcirk030428	99	1.71	19.1	21.2	1.27	152	127	73	35	0.48
300Wcirkbox030502	295	1.79	19.1	21.6	2.73	341	126	80	32	7.75
1000cirkbox030507	982	1.90	19.2	21.9	7.75	1,025	124	81	29	26.17
50cirkbox030508	49	1.68	19.7	21.4	0.92	108	128	69	40	31.96
100cirkbox030513	110	1.82	19.6	22.0	1.26	160	127	78	33	2.76
200cirkbox030516	193	1.72	19.8	21.6	2.07	248	127	72	40	27.54
400cirkbox030519	392	1.90	19.8	21.8	3.33	442	126	76	38	10.68
250cirkbox030520	247	1.83	19.8	21.8	2.37	301	127	73	38	14.22
355cirkbox030521	349	1.88	19.9	21.8	3.05	400	126	75	38	11.11
453cirkbox030522	445	1.86	19.8	21.9	3.81	493	126	76	38	12.24
550cirkbox030524	539	1.84	19.8	21.8	4.61	591	126	74	36	3.10
500cirkbox030601	494	1.90	19.7	21.8	4.17	553	125	65	31	9.80
150W1cirkbox030527	147	1.79	19.8	21.7	1.64	204	127	69	36	1.20
									mean	Total Difference^2
									34.48	224.76
										Standard deviation
										3.00

The heat transfer coefficient is assumed to be constant in the prevailing temperature range. The mean value is calculated to be 34.5 W/°C with a standard deviation of ± 3 W/°C.

5.2.2 Calculation of a fuel assembly’s decay heat power by means of circulation calibration

- The fuel assembly is placed in the calorimeter.
- A flow through the calorimeter is created where surrounding pool water is pumped into the calorimeter.
- The fuel assembly brings the hot water in the calorimeter to a state of equilibrium with respect to the temperature of the pumped-in water and the pumped-out water.

- All measurement data are logged at this state of equilibrium.
- The heat output that is carried away with the flow (Flow power) through the calorimeter is calculated by $E = F \times cp \times \Delta T$, where E is the power (Watts), F is the flow (kg/s), cp is the heat capacity of water J/(kg×s) and ΔT is the difference between the temperature of the incoming water (KA501) and the temperature of the outgoing water (KA502).
- The heat that is transported through the calorimeter wall E_k is calculated by $E_k = KA \times \Delta T_k$.
- The fuel assembly's decay heat power is calculated by adding the flow power and the leakage power E_k and then subtracting the pump power (PA2).
- If the temperature increases or decreases during measurement, the amount of energy absorbed/emitted by the water, the fuel and the steel in the calorimeter is calculated. The effect of the temperature increase/decrease is rarely greater than 1 Watt.
- No compensation is made for the volume difference between the electric heater and the fuel assembly, since compensation is made on the deviation from the state of equilibrium, which is rarely greater than 1 Watt.

Table 5-7 shows calculations of the heat output for the three fuel assemblies that were measured in the summer of 2003.

The accuracy ± 6 Watts is based on the standard deviation for KA, which is ± 3 Watts/°C.

It should be pointed out that it is important in connection with calibration and particularly fuel measurement that the selected temperature difference be set as exactly as possible. In the event of deviation from the selected temperature difference, a measurement error will be introduced due to the fact that it is not known how large a percentage of the pump power is converted to heat in the calorimeter. With the calculation method that has been used, the entire pump power is added to the power that heats the water and enters into the heat transfer coefficient, KA, as the term $P/\Delta T_k$, where P is the pump power and ΔT_k is the temperature difference during calibration. Then during fuel measurement the heat transfer coefficient KA is multiplied by ΔT_m (the temperature difference during calibration), and thereby also by $P/\Delta T_k$. If ΔT_k and ΔT_m are not equal during calibration and measurement, an error will be introduced which is $\Delta T_m/\Delta T_k \times (P - P_{\text{right}})$ where P_{right} is the fraction of the pump power that heats the water in the calorimeter. It can be seen in Tables 5-6 and 5-7 that it is possible to reduce the difference in the temperature differences prior to the next calibration and fuel measurement and thereby improve accuracy.

Table 5-7. Heat output for 3 fuels.

Fuel	Date	Bulk temp °C	PA2 W	Flow kg/min	DT °C	Tmc °C	Flow power W	Leakage power W	Assembly's decay heat power W
9329	30602	19.68	126.7	2.23	1.79	21.70	279	70	220 ± 6
9329	30605	19.67	127.5	2.21	1.80	21.85	277	75	227 ± 6
6432	30607	19.79	127.5	1.83	1.93	21.88	246	72	190 ± 6
6432	30611	19.81	128.0	1.86	1.90	21.87	246	71	190 ± 6
5A3	30614	19.71	127.6	2.34	1.82	21.68	297	68	237 ± 6
5A3	30617	19.75	127.7	2.34	1.83	21.72	298	68	239 ± 6
5A3	30619	19.32	127.6	2.34	1.83	21.27	299	67	238 ± 6

5.3 Heat losses due to gamma radiation

Some of the decay energy from the fuel assembly is lost with gamma radiation that is not stopped in the calorimeter, called gamma escape. The above-calculated decay heat power must be corrected for the energy quantity that is lost with the gamma radiation. To do this, the gamma radiation from the fuel is measured on the outside of the calorimeter with 5 gamma probes at a distance from the centre as given in Table 5-8 below.

The gamma arm on which the gamma probes are mounted is positioned approximately 1.7 m from the bottom of the fuel.

In ref. /2/ a relationship has been determined between the gamma escape power P_γ and the measured gamma dose rate (Gy/h) d_i based on the geometric configuration of the gamma probes. The expression is as follows.

$$P_\gamma = a \frac{\sum_{i=1}^5 d_i e^{-\lambda r_i}}{\sum_{i=1}^5 (e^{-\lambda r_i})^2} \left\{ e^{-\lambda} \left(-\frac{1}{\lambda^2} - \frac{1}{\lambda} \right) + e^{-0.228\lambda} \left(\frac{1}{\lambda^2} + \frac{0.228}{\lambda} \right) \right\} F \quad \text{Equation 5-1}$$

$a = 8.14$ W for BWR fuel and 8.41 W for PWR fuel.

$\lambda = 8.4 \text{ m}^{-1}$ the attenuation coefficient.

$r_i =$ the distance of the gamma probes from the centre as shown in Table 5-8.

$F =$ the profile factor, which is unique for each fuel. Appendix 5 gives F factors for a number of fuel assemblies.

For each fuel measurement, the gamma dose rates are logged with the probes d_1 – d_5 . The dose rates are inserted in the above expression and the gamma escape power P_γ can be calculated. The dose rates for the five probes d_1 – d_5 for the three assemblies 9329, 6432 and 5A3 are given in Table 5-9.

Table 5-8. Distance between centre and gamma probe.

Probe d_i	Radius (m)
1	0.26
2	0.38
3	0.50
4	0.63
5	0.77

With the data in Table 5-9, Equation 5-1 gives the gamma escape power P_γ , which is shown in Table 5-10.

Table 5-9. The dose rates for the five probes d_1 – d_5 for the three assemblies 9329, 6432 and 5A3.

Fuel	Probe d_i (Gy/h)				
	1	2	3	4	5
9329 (030602)	5.86	3.71	1.81	0.75	0.39
9329 (030604)	5.92	3.76	1.83	0.75	0.39
6432 (030610)	6.84	3.86	1.77	0.74	–0.04
6432 (030612)	7.10	3.94	1.76	0.74	0.38
5A3 (040427)	12.50	4.88	1.49	0.50	0.20

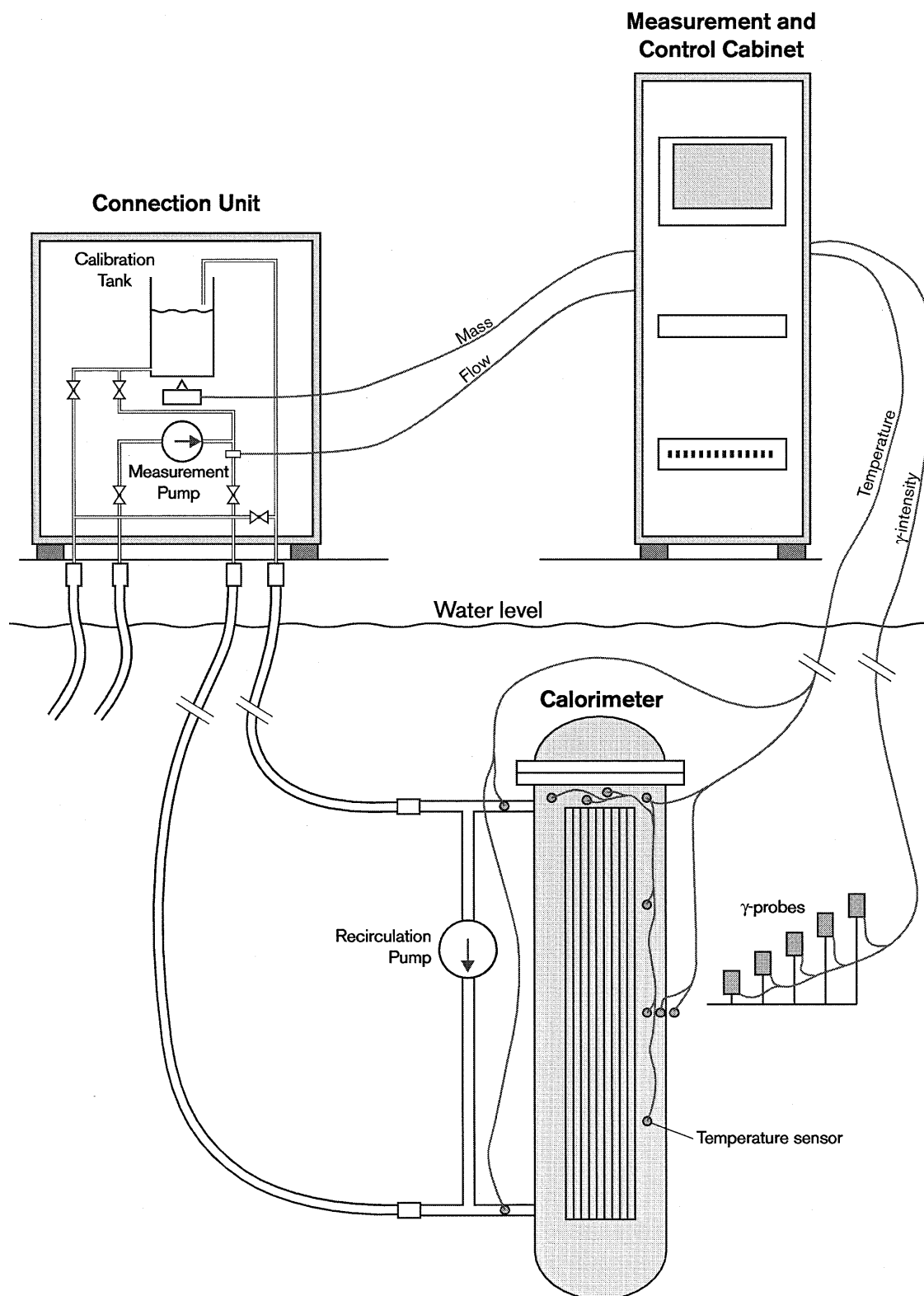
Table 5-10.

Fuel	Gamma escape power P_γ (Watts)
9329 (030602)	2.8
9329 (030604)	2.8
6432 (030610)	3.1
6432 (030612)	3.2
5A3 (040427)	5.8

6 References

- /1/ Systembeskrivning Clab System 251.
- /2/ Rapport angående avgiven gammaeffekt i BWR- och PWR-bränsle. Department of Radiation Sciences Uppsala reg nr 2004-05604.

Illustration of calorimeter system 251



Primary data temperature increase method

Tinfo1	Tinfo2	Logg_tid	T1	T2	T3	T4	T5	T6	T7	T8	T9	T10	T11	T12	T13
Fredrik Sturek	150Wtempstegbox03052														
KA701	KA702														
360	16														
0,00424	0,00421														
KA703	KA704														
76	-514														
0,00449	0,00425														
KA705	CALIB. TEMP														
-132	21,5														
0,00451	AMPLIFICATION														
	023														
			_251KA901_A_K	_251KA509	_251KA501_B1	_251KA501_B2	_251KA502_B1	_251KA502_B2	_251KA511_B1	_251KA511_B2	_251KA511_B3	_251KA511_B4	_251KA511_B5	_251KA511_B6	_251KA511_B7
			W	°C	°C	°C	°C	°C	°C	°C	°C	°C	°C	°C	°C
Mon May 26 2003 08:30:22		80	149.52												
		96		0,160	18,067	18,127	18,276	18,279							
		120	149,81												
		180	149,73												
		192		-0,191	18,452	18,470	18,268	18,273	18,252	18,244	18,234	18,235	18,162	18,184	18,256
		240	150,10												
		288		-0,307	18,567	18,593	18,271	18,274							
		300	150,16												
		360	150,16												
		364		-0,475	18,733	18,760	18,269	18,275	18,207	18,232	18,241	18,251	18,225	18,252	18,283
		420	149,73												
		480	149,90		-0,613	18,896	18,918	18,291	18,297						
		540	149,69												
		576		-0,728	19,029	19,048	18,307	18,313	18,271	18,290	18,262	18,268	18,276	18,291	18,306
		600	149,78												
		660	149,24												
		672		-0,821	19,134	19,155	18,321	18,325							
		720	149,49												
		768		-0,684	19,215	19,229	18,337	18,338	18,297	18,322	18,310	18,317	18,305	18,318	18,332
		780	149,95												
		840	150,41												
		864		-0,934	19,280	19,294	18,351	18,355							
		900	150,13												
		960	150,32		-0,971	19,332	19,342	18,364	18,368	18,331	18,348	18,341	18,348	18,330	18,347
		1020	149,89												
		1056		-1,001	19,378	19,386	18,379	18,383							
		1080	150,41												
		1140	150,36												
		1152		-1,022	19,410	19,423	18,391	18,397	18,355	18,379	18,370	18,376	18,357	18,374	18,391
		1200	150,30												
		1248		-1,044	19,444	19,459	18,404	18,411							
		1280	150,36												
		1320	150,93												
		1344		-1,053	19,470	19,482	18,422	18,425	18,383	18,404	18,396	18,403	18,390	18,404	18,416
		1380	150,51												
		1440	149,54		-1,063	19,495	19,506	18,436	18,439						
		1500	149,49												
		1536		-1,070	19,515	19,528	18,449	18,453	18,413	18,431	18,425	18,435	18,416	18,432	18,448
		1590	149,42												
		1620	149,84												
		1632		-1,076	19,535	19,547	18,465	18,466							
		1680	149,87												
		1728		-1,078	19,553	19,564	18,478	18,483	18,444	18,462	18,453	18,464	18,443	18,458	18,475
		1740	149,74												
		1800	149,57												
		1824		-1,080	19,568	19,579	18,492	18,495							
		1880	149,72												
		1920	150,43		-1,080	19,562	19,594	18,506	18,510	18,469	18,467	18,463	18,490	18,471	18,487
		1980	149,78												
		2016		-1,078	19,595	19,607	18,520	18,526							
		2040	149,97												
		2100	149,57												
		2112		-1,076	19,607	19,618	18,534	18,540	18,499	18,514	18,514	18,520	18,502	18,518	18,534
		2160	149,13												
		2208		-1,070	19,616	19,627	18,550	18,554							
		2220	149,13												
		2280	149,54												
		2304		-1,064	19,624	19,635	18,564	18,568	18,528	18,540	18,538	18,546	18,530	18,545	18,563
		2340	149,51												
		2400	149,89		-1,056	19,632	19,643	18,579	18,585						

T14	T15	T16	T17	T18	T19	T20	T21	T22	T23	T24	T25	T26	T27
_251KA511_B8 °C	_251KA511 °C	_251KA751_Vikt kg	_251KA101 bar	_251KA521_B1 °C	_251KA522_B1 °C	_251KA531_B1 °C	_251KA531_B2 °C	_251KA701 Gy/h	_251KA702 Gy/h	_251KA703 Gy/h	_251KA704 Gy/h	_251KA705 Gy/h	_251KA901_B_P W
						19.743	19.749						
18.196	18.223			18.218	19.741	19.752	19.751						
		11.17				19.749	19.746						
18.254	18.243			18.248	19.746	19.745	19.751						
						19.748	19.747						
18.287	18.286	11.17	1.39	18.280	19.747	19.746	19.746	-0.22	-0.10	-0.08	-0.10	-0.04	127.97
						19.746	19.748						
18.318	18.315			18.310	19.745	19.746	19.748						
		11.17				19.739	19.749						
18.343	18.344			18.342	19.741	19.744	19.747						
						19.746	19.742						
18.368	18.371	11.17	1.37	18.368	19.742	19.746	19.747	-0.22	-0.10	-0.06	-0.10	-0.04	127.85
						19.746	19.749						
18.402	18.400			18.397	19.741	19.746	19.748						
		11.17				19.746	19.746						
18.427	18.428			18.425	19.739	19.745	19.749						
						19.747	19.750						
18.455	18.457	11.18	1.38	18.455	19.740	19.747	19.749	-0.28	-0.17	-0.18	-0.18	-0.15	127.57
						19.746	19.747						
18.487	18.485			18.485	19.739	19.747	19.749						
						19.747	19.751						
18.524	18.516	11.18		18.512	19.737	19.747	19.747						
						19.746	19.749						
18.544	18.542	11.19	1.37	18.527	19.738	19.746	19.748	-0.25	-0.14	-0.13	-0.14	-0.10	127.52
						19.747	19.751						

T14	T15	T16	T17	T18	T19	T20	T21	T22	T23	T24	T25	T26	T27	T28	T29	T30
_251KA511_B7 °C	_251KA511_B8 °C	_251KA511 °C	_251KA751_Vikt kg	_251KA101 bar	_251KA521_B1 °C	_251KA522_B1 °C	_251KA531_B1 °C	_251KA531_B2 °C	_251KA701 Gy/h	_251KA702 Gy/h	_251KA703 Gy/h	_251KA704 Gy/h	_251KA705 Gy/h	_251KA901_B_P W	_251KA111 bar	_251KA112 bar
21.194	21.197	21.199														
			0,00													
21.201	21.211	21.209														
21.212	21.217	21.217	0,01	1.53					-0.20	-0.07	-0.02	-0.08	-0.00	127.36	0.55	0.37
					21.207	19.762	19.745	19.745								
21.215	21.222	21.224														
			0,00													
21.222	21.228	21.230														
21.229	21.233	21.236	0,00	1.53					-0.25	-0.14	-0.13	-0.14	-0.09	127.09	0.54	0.38
21.235	21.241	21.242			21.229	19.762	19.749	19.750								
			-0,01													
21.242	21.248	21.249														
21.248	21.256	21.256														
			0,00	1.54					-0.13	0.02	0.11	0.01	0.10	126.56	0.55	0.38

Example of calculation of KA in connection with calibration according to circulation method

Circulation method calibration.

Mean flow; F (kg/min)	1.459 kg/min
Mean DT; °C	2.016°C
T _{pool} mean	18.455°C
T _{mc} mean	20.616°C
Mean power from heater (Watts)	147.247 W
Volume of calorimeter (litres)	389.8
Volume of electric heater without guide (litres)	13.3 litres
H ₂ O quantity in calorimeter (litres)	376.5 litres
Density H ₂ O in calorimeter kg/m ³	998.27843 kg/m ³
Enthalpy for H ₂ O in calorimeter at Dt _{start}	86.095739 KJ/kg
Enthalpy for H ₂ O in calorimeter at Dt _{stop}	85.983561 KJ/kg
Pressure at lower part of calorimeter	2.5 bar
Pressure at upper part of calorimeter	2 bar
C _p in calorimeter	4.1810906 kJ/kg°C
C _p for steel	0.46 kJ/kg°C
C _p for copper	0.385 kJ/kg°C
C _p for PTFE	1 kJ/kg°C

Flow power (Watts): q₁	204.993 Watt
q ₁ =F×DT×c _p	
/Watt = W/	
Pump power	127.53 Watt
Total supplied power: q_{tot}	274.773 Watt
Energy change H₂O	
equ for DT -5,61156E-07×2,09661E+00	
start DT (°C)	2.029
stop DT (°C)	2.002
energy	-42.162 KJ
during period /sek = s/	4,7811.0 sek
Power	-0.882 Watt
Energy change steel liner	
c _p ×(DT _{start} -DT _{stop})×m	-3.642 KJ
during period	4,7811 sek
Power	-0.076 Watt
Energy change copper wire	
c _p ×(DT _{start} -DT _{stop})×m	-0.036 KJ
during period	4,7811 sek
Power	-0.0008 Watt

Energy change PTFE	
$cp \times (DT_{start} - DT_{stop}) \times m$	-0.231 KJ
during period	4,7811 sek
Power	-0.0048 Watt
Energy change total:q2	-0.9636 Watt
Heat leakage :q3	70.7444 Watt
KA coefficient	32.732035 W/°C

F factors for calculation of gamma escape power**Table of F factors for calculation of gamma escape power.**

2014	0.975622
2018	0.914954
2048	0.962891
2074	0.955906
2118	0.881336
9329	0.994241
10288	1.035675
14076	0.970250
3838	1.038405
KU0100	1.012166
KU0269	0.897371
KU0278	0.969331
KU0282	0.937214
5535	1.146337
11494	1.052320
11495	1.022379
13775	1.014151
13847	1.007282
13848	1.000800
12078	0.995409
13628	1.024523
13630	1.001697
1377	1.081402
1389	0.880393
1546	0.988952
1696	1.015446
1704	0.955050
2995	0.945833
3054	1.006630
3058	0.990371
3064	0.968811
6350	1.011223
12684	0.985654
582	0.991357
596	1.013750
710	0.982377
900	0.995106
1136	1.009810
1177	1.015357
1186	0.969020
5829	0.987186
6423	0.964806
6432	0.964992
6454	0.913335
6478	0.919575

8327	0.951297
8331	0.986884
8332	1.003705
8338	0.938861
8341	0.979672
8327	0.977504
6423	0.967928
6478	0.752138
A05	0.993411
A11	0.973324
C01	1.100458
C12	1.011120
C20	1.030648
C42	1.047301
D27	0.959770
D38	1.001761
E38	0.961359
E40	1.005867
F14	0.998639
F21	0.972326
F25	1.020216
F32	1.028738
G11	0.995221
G23	0.993674
I09	1.059126
I20	0.956234
I24	1.054825
I25	0.974561
E38	0.984836
2A5	1.030900
5A3	1.018698
0C9	0.977004
1C2	1.032726
1C5	0.999136
2C2	1.003833
3C1	1.050024
3C4	1.097825
3C5	1.025466
3C9	0.941833
4C4	1.000587
4C7	1.022885
0E5	1.032234
0E6	1.028331
1E5	1.035667
5F2	0.975622

Clab – Uncertainty analysis of the calorimeter (system 251) using the temperature increase method

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Contents

1	Introduction	139
2	General about uncertainties	141
3	Procedure for measurement and evaluation	143
4	Uncertainties	145
5	Determination of uncertainties	149
6	Conclusion	155
	References	157
	Enclosure 1	159
	Enclosure 2 Simplification of the formula for P_γ	161

1 Introduction

The design target for the calorimeter is that decay heat power should be determined with the uncertainty of $\pm 2\%$.

The interpretation of this is that the real decay heat power should be within -2% till $+2\%$ of the measured value with 95% confidence. If for example the measurement shows that the decay power an assembly is 200 W the probability should be 95% that the real power is between 196 W and 204 W.

The design target is high and even if it shows the target not is met within the whole measurement range 50–1,000 W this high ambition has contributed to the fact that the equipment has been designed so the uncertainties will be as small as possible with reasonable means.

As comparison it could be mentioned that the GE-Morris equipment, which has served as basis for the Clab-calorimeter gave an uncertainty of ± 15 W (two standard deviations). This corresponds to $\pm 7.5\%$ with 95% confidence at 200 W.

The purpose with this report is to determine the uncertainty in the measured decay heat power.

2 General about uncertainties

Uncertainties in the measured value could come from variations (errors) in the measurement chains and equipment. These errors could be separated into three categories which are systematic, variable and random /1, 2/.

- Systematic errors are constant errors which will not change at repeated measurements of the same measurement point.
- Variable errors will change systematically at repeated measurements of the same measurement point.
- Random errors will show up at random at repeated measurements of the same measurement point.

In this investigation the errors that occur at random and variable errors are analysed. Variable errors are interpreted as errors that occur as result of different measurement conditions at repeated measurements.

Systematic errors are not analysed in this work.

3 Procedure for measurement and evaluation

In part 3 the procedure for developing a calibration curve for the equipment is described.

In summary the procedure is:

1. The electric heater is installed in the calorimeter and adjusted to a predetermined power.
2. The calorimeter is cooled down by circulating water from the deionised water system, which has lower temperature than the pool water.
3. After the circulation is stopped logging of the temperatures, time and other parameters is started.
4. Logging is stopped after a certain temperature increase

The temperature difference between the calorimeter and the pool water is calculated by subtracting the average value of the temperature sensors inside with average value of the temperature sensors outside the calorimeter. Presently two sensors inside and two sensors outside the calorimeter are used.

A second degree curve is correlated to the temperature difference and time. The slope of the curve at the zero temperature difference is given by the coefficient of the first degree term in the equation.

The procedure is repeated at several different powers in the electric heater. The result will be a table with slope values and corresponding power values. From this table a calibration curve is generated by correlating a straight line to the slope and power values. The correlation gives the equation of this straight line.

When measuring a fuel assembly the following procedure is followed:

1. An assembly is placed in the calorimeter (the heater is removed before).
2. The calorimeter is cooled down with deionised water.
3. After the circulation is stopped logging of the temperatures, time, gamma radiation rate and other parameters is started.
4. Logging is stopped after a certain temperature increase.

The temperature difference between the calorimeter and the pool is calculated by subtracting the average value of the temperature sensors inside with average value of the temperature sensors outside the calorimeter. Presently two sensors inside and two sensors outside the calorimeter are used.

A second degree curve is correlated to the temperature difference and time. The slope of the curve at the zero temperature difference is given by the coefficient of the first degree term in the equation.

A correction of the slope is done to account for the fact that the fuel assembly has different volume and contains different materials than the electric heater.

The resulting slope is used in the calibration curve equation, which is solved for the power.

With the measured gamma dose rates the power that escapes the calorimeter with the gamma radiation is calculated using the equation given in part 3.

The sum of the power in the calorimeter and the power corresponding to the gamma field will give the decay heat power for the measured fuel assembly.

4 Uncertainties

In the system description of system no 251 /3/ the following error limits are reported for the measurement equipment:

Temperature	$\pm 0.01^{\circ}\text{C}$
Massflow	$\pm 0.5\%$ in the rang 0.36 kg/min–7.2 kg/min
Power	$\pm 0.25\%$ in the range 50 W–1,000 W
Gamma dose rate	$\pm 5\%$ in the range 0–100 Gy/h
Mass	$\pm 0.02\%$

These errors contribute to the random errors in the measurements.

To identify the parameters which can give uncertainties a basic, simplified heat balance could be used. The following notations are used:

Total power in the calorimeter: P
 Power in the circulation pump: P_{pump}
 Efficiency of pump: η
 Heat loss from calorimeter with gamma radiation: P_{gamma}
 Heat loss through the calorimeter: P_{loss}
 Heat loss due to heating up the calorimeter with inserts: P_{cp}
 Measured temperatures in calorimeter: $t_{\text{KA502B1}}, t_{\text{KA502B2}}$
 Measured temperature in the pool: $t_{\text{KA531B1}}, t_{\text{KA531B2}}$
 Measured gamma dose rates: $d_{\text{KA701}}, d_{\text{KA702}}, d_{\text{KA703}}, d_{\text{KA704}}, d_{\text{KA705}}$
 Measured time points: T_1, T_2

A heat balance for the calorimeter will look as follows:

$$P + \eta P_{\text{pump}} = P_{\text{loss}} + P_{\text{cp}} + P_{\text{gamma}} \text{ where}$$

P_{pump} = measured value of the pump power
 η = pump efficiency is not known

$P_{\text{loss}} = kA(t_{\text{cal}} - t_{\text{pool}})$, where k is the aggregate heat transfer coefficient of the calorimeter, A is the calorimeter surface area, the temperature in the calorimeter is $t_{\text{cal}} = 0.5(t_{\text{KA501B1}} + t_{\text{KA502B2}})$ and $t_{\text{pool}} = 0.5(t_{\text{KA531B1}} + t_{\text{KA531B2}})$ is temperature in the pool. Here the aggregate heat transfer coefficient k is unknown.

$P_{\text{cp}} = \sum (mc_p)_i dt/dT$ that is the sum of mass times latent heat for all different materials in the calorimeter times the rate of temperature increase. This term can be calculated.

$P_{\text{gamma}} = 0.378d_{\text{KA701}} + 0.138d_{\text{KA702}} + 0.050d_{\text{KA703}} + 0.017d_{\text{KA704}} + 0.005d_{\text{KA705}}$ is the power generated in the fuel but escapes from the calorimeter with the gamma radiation. This is a development of the formula given in part 3 to calculate P_{gamma} , see enclosure 2. The coefficients in front of each measured dose rate is calculated from parameter which correlates the measured dose rate values to an exponential function.

The heat balance will be:

$$P = kA(t_{\text{cal}} - t_{\text{pool}}) + \sum (mc_p)_i dt/dT + P_{\text{gamma}} - \eta P_{\text{pump}}$$

And the rate of temperature increase:

$$dt/dT = (P + \eta P_{\text{pump}} - kA(t_{\text{cal}} - t_{\text{pool}}) - P_{\text{gamma}}) / \Sigma mc_p$$

It can be seen that during calibration the uncertainty is controlled by uncertainties in power in the electric heater, heat from the pump, heat losses from the calorimeter and the material in the calorimeter.

When measuring a fuel element the uncertainties are governed by uncertainties in heat from the pump, heat losses from the calorimeter, the material in the calorimeter and determination of the gamma power.

The temperature increase is calculated with help of four temperature sensors each with an uncertainty of $\pm 0.01^\circ\text{C}$ which gives a total uncertainty $\pm 0.02^\circ\text{C} \sqrt{(4 \times 0.01^2)}$.

The power in the electric heater is known and regulated with a specified accuracy of $\pm 0.25\%$, which corresponds to $\pm 0.5 \text{ W}$ at 200 W . The reading of the power is corrected to account for the heat loss in the electric heater feed cable.

The gamma field outside the calorimeter is measured by five gamma probes. The gamma dose rate is measured with an accuracy of $\pm 5\%$. Five probes give a total uncertainty around $\pm 10\%$.

The heat losses from the calorimeter depends on the calorimeter surface area, which is constant and the heat transfer coefficient which in turn depends on temperature conditions, geometries, flow conditions and temperature in the surrounding pool water. Uncertainties here are generated by variations in flow conditions because the heater and the fuel assembly placing in the calorimeter are different at different measurements and also by variations in circulation flow and variations in the pool temperature.

The power from the circulation pump is constant and monitored continuously. Added heat to the calorimeter is controlled by the efficiency, which is unknown. This contributes to the uncertainty.

The material in the calorimeter consists of the calorimeter itself, inner frame work, inserts, water, the heater or a fuel assembly. These are known.

Of this discussion follows that following parameters that can be quantified gives contributions to the uncertainties.

- Measurement uncertainties of the power in the electric heater, of the power in the circulation pump, of temperatures and gamma dose rate.
- Uncertainties in materials and volumes of the heater and the fuel elements, including length increase due to irradiation.

Other parameters for which uncertainties not can be quantified separately today:

- Variations in flow and temperature conditions in the calorimeter, which influence the heat transfer from the fuel rods to the water in the calorimeter and the heat transport from the calorimeter the outside. Differences in the designs of heater and fuel assemblies and different designs of fuel assemblies can give different flow conditions in the calorimeter. This uncertainty is minimized two ways, the temperature in the calorimeter is equalized by the circulation flow and the measurement is performed at zero temperature difference between the calorimeter and the pool water. The heat transfer from the fuel rods to the water in the calorimeter is, however, influenced by different fuel assembly designs (box/no box, BWR/PWR, water channel/no water channel and others).

- Variations in the pool water temperature are monitored but the influence on the measured result has not been quantified and this effect has to be treated as an uncertainty.
- Variations in pool water temperature are monitored but the influence on the measured result has not been quantified and this effect has to be treated as an uncertainty
- Axial (vertical) variations in temperature along the calorimeter has been measured /4/ and found small. Thos effect is judged not to contribute to the uncertainty.

5 Determination of uncertainties

When important parameters in the calorimeter are unknown an analytical determination of the uncertainties can not be done. It remains to assess these by experiment, for example by measure the reproducibility by measuring the same power several times and study the spread of the measurements or study the spread of a series of measurements.

Uncertainty in one single measurement

When developing calibration curves measurements have been done for a large number of cases at different powers. In each case the temperature is monitored during a long time period. For one case at 150 W the temperature was logged 150 times during 4 hours. Then the temperature had increased 2°C. The measurement series is adjusted so that the time is zero when the temperature difference is zero. This means that the measurement series starts at – 7,200 sec and –1°C and stops at 7,200 sec and 1°C. A correlation of the series to a second degree curve is done. In this equation the coefficient before the first degree term is the wanted slope value. The standard deviation in the temperature valued can be computed to 0.005°C which is a very low value. Based on this the standard deviation of the slope value is judged to be very small. The uncertainty in the in fitting of the second degree curve the measured points is judged to give a negligible contribution to the uncertainty.

Uncertainty in repeated measurements

The uncertainty can be determined experimentally by measuring the slope value several times at the same power. In the measurement series there are three measurement points at 147.3 W which gave 1.3494E–04, 1.3837E–04 respective 1.3874E–04°C/s. The difference between the highest and lowest value is 3% which give an indication of the reproducibility and the uncertainty.

Uncertainty in a calibration curve

A large number of measurements are done at different powers to get information to produce a calibration curve. This curve is produced by fitting a strait line by the last square method to the all the slope values and powers. The general equation is then

$$P_{\text{cal}} = a + b \, dt/dT \text{ where}$$

P_{cal} = power in the heater at calibration

a and b = constants from the curve fitting

dt/dT = measured rate of temperature increase

In part 3 fittings of a strait line is done for the measured points in power between 50–350 W. The fit gives the following:

$$P_{\text{cal}} = 1.73 \times 10^6 \times dt/dT - 90.0, \text{ this equation is used for evaluation of BWR -fuel.}$$

The standard deviations (s) in the constants in the equations has been calculated to $s_b = 6,938 \text{ W/s}^\circ\text{C}$ respective $s_a = 1.9 \text{ W}$ according to /1/, see enclosure 1.

For the PWR-assemblies in the measurement program the powers are expected to be in the range of 250 and 900 W. This gives the equation:

$P_{cal} = 1.72 \times 10^6 \times dt/dT - 86.4$. The standard deviations (s) in the constants in the equations has been calculated to $s_b = 6,442 \text{ W s}/^\circ\text{C}$ respective $s_a = 3.5 \text{ W}$ according to /1/, see enclosure 1.

Uncertainty in measuring one fuel assembly

When measuring the decay heat power of one fuel assembly the temperature increase rate is determined (dt/dT). This value has to be corrected with a constant (k) reflecting the difference between the materials and volumes between the heater and the fuel assembly.

This constant is calculated according a procedure in part 3 and summarized in Table 5-1 below:

Table 5-1. Volumes and materials in the calorimeter, heater and typical fuel.

		Volumes with heater		Volumes BWR		Volumes PWR	
		no box	box	no box	box	15x15	17x17
Volumes	Vol (dm ³)						
Mätcylinder med fackverk och BWR instyrning	389.80	389.80	389.80	389.80	389.80		
Mätcylinder med fackverk						390.20	390.20
Fackverk	5.25						
BWR-instyrning	0.40						
Distans	1.41			1.41			
Elvärmare	13.30	13.30	13.30				
Plåtar till elvärmare	2.27		2.27				
Instyrning	0.51	0.51	0.51				
Boxat BWR	35.60				35.78		
Oboxat BWR	30.10			28.61			
PWR-element	85.8					75.41	76.56
Free water (dm³)		375.99	373.72	359.78	354.02	314.79	313.64
Mass steel in calorimeter							
Mantel (kg)		112.33	112.33	112.33	112.33	112.33	112.33
Botten (kg)		2.15	2.15	2.15	2.15	2.15	2.15
Dysa (kg)		48.30	48.30	48.30	48.30	48.30	48.30
Lock (kg)				3.42	3.42	3.42	3.42
Cirkledning (kg)		5.84	5.84	5.84	5.84	5.84	5.84
Fackverk (kg)		42.03	42.03	42.03	42.03	42.03	42.03
BWR-instyrning (kg)		3.23	3.23	3.23	3.23		
Distans (kg)				10.90			
Mass steel in heater (kg)							
Värmarlock (kg)		5.38	5.38				
Instyrning (kg)		4.11	4.11				
Plåtar (kg)			18.60				
Total mass steel (kg)		295.07	313.67	228.20	217.30	214.07	214.07

Mass material in calorimeter	Volumes with heater		Volumes BWR		Volumes PWR		
	no box	box	no box	box	15x15	17x17	
Mängd stål i kalorimetern (kg)	295.07	313.67	228.20	217.30	214.07	214.07	
Mängd vatten i kalorimetern (kg)	375.99	373.72	359.78	354.02	314.79	313.64	
Mängd Cu i värmaren (kg)	3.48	3.48					
Mängd plast i värmaren(kg)	8.60	8.60					
Mängd Zr i bränslet (kg)			47.62	81.39	123.75	117.59	
Mängd stål/inc i bränslet (kg)			4.20	9.75	17.83	20.55	
Mängd UO ₂ i bränslet (kg)			196.7931733	201.8064	499.98	521.33	
	Cp						
MCp för stål i kalorimetern (kJ)	0.46	135.73	144.29	104.97	99.96	98.47	98.47
MCp för vatten i kalorimetern (kJ)	4.18	1,571.64	1,562.15	1,503.89	1,479.81	1,315.83	1,311.03
MCp för Cu i värmaren (kJ)	0.40	1.39	1.39				
MCp för plast i värmaren(kJ)	1.00	8.60	8.60				
MCp för Zr i bränslet (kJ)	0.28			13.33	22.79	34.65	32.93
MCp för stål/inc i bränslet (kJ)	0.46			1.93	4.48	8.20	9.45
MCp för UO ₂ i bränslet (kJ)	0.23			45.26	46.42	114.99	119.90
Summa MCp		1,717.36	1,716.43	1,669.39	1,653.46	1,572.15	1,571.79
k			0.9995	0.9721	0.9628	0.9154	0.9152

The volume of water and steel and the volume of the heater are determined by experiment, see /5/. Uncertainties in these values are small and could be neglected.

An uncertainty will be introduced when the heater is replaced by a fuel assembly. The volume value of a fuel assembly is received from the power plant or the fuel manufacturer and contains a certain uncertainty.

This uncertainty will be analyzed for all the fuel assemblies in the measurement program. The volume of a BWR assembly is around 36 dm³. Assume that this value has an error of 3 dm³. If the volume is overestimated water will be replaced by fuel material, the total volume being constant. If Zr, SS and UO₂ are increased with 3 dm³ the k will increase with a factor of 0.003 or 0.3%. An error in the fuel volume gives a relatively small error in the measured power. For PWR a 10% error in the volume estimation will give an error of 0.7% in the measured power.

An error in the fuel volume gives a relatively small error in the measured power.

Uncertainty in P_{gamma}

P_{gamma} is calculated with the equation $0.378d_{\text{KA701}}+0.138d_{\text{KA702}}+0.050d_{\text{KA703}}+0.017d_{\text{KA704}}+0.005d_{\text{KA705}}$. The uncertainty in P_{gamma} depends on the uncertainty in the parameters that are included in the constants before each measured dose rate value

There are presently not enough data to calculate these uncertainties based on experimental data. Therefore the uncertainty in P_{gamma} is estimated based on part 3. There P_{gamma} for assembly 9329 is calculated to 5.2 W with an error of ± 0.48 W or 9%. The calorimetric measured power was 217.4 W and the total decay heat power is then 232 W. P_{gamma} is 2.3% of P and the error in P_{gamma} corresponds to 0.2% of the total power. For now it is assumed that P_{gamma} is proportional to the calorimetric power.

Uncertainty in the power of a fuel assembly

The complete equation which will give the power from the temperature increase rate is:

$$P = P_{\text{cal}} + P_{\text{gamma}} = (a + b k dt/dT + 0.378d_{\text{KA701}} + 0.138d_{\text{KA702}} + 0.050d_{\text{KA703}} + 0.017d_{\text{KA704}} + 0.005d_{\text{KA705}}).$$

If P_{gamma} is assumed to be proportional to P_{cal} the equation will be

$$P = P_{\text{cal}} + P_{\text{gamma}} = P_{\text{cal}}(1 + \alpha)$$

where α is the fraction power that will disappear from the calorimeter with the gamma radiation. This gives:

$$P = (a + b k dt/dT)(1 + \alpha)$$

The constants are summarized below

Constants	BWR (with box)	PWR
a	-89.98	-86.36
b	1,729,644	1,716,819
k	0.963	0.915
α	0.024	0.024

The errors in the constants (standard deviations) are summarized:

Parameter	BWR	PWR
s_a	1.9	3.5
s_b	6,938.0	6,442.0
s_k	0.0030	0.0070
s_α	0.002	0.002

In order to get the standard deviation in P the standard deviations of the parameters are combined as follows according to the method in /2/:

$$s_p = \sqrt{[(s_a(1+\alpha))^2 + (s_b k(1+\alpha) dt/dT)^2 + (s_k b(1+\alpha) dt/dT)^2 + (s_\alpha b k dt/dT)^2]}$$

The standard deviation has to be calculated for each measurement. As example a calculation is of the standard deviation is done for two measurements, for one BWR-assembly and for one PWR-assembly.

In the BWR-case $dt/dT = 1.8464E-4^\circ\text{C/s}$ was measured for assembly 9329. According to the equation above the total power is calculated to $P = 222.6\text{W}$, $P = (-89.98 + 1.8464E-4 \times 1.73E6 \times 0.963)(1 + 0.024)$.

The standard deviation is calculated to $\pm 2.6\text{ W}$ [$s_p = \sqrt{(1.9 \times 1.024)^2 + (6,938 \times 1.8464E-4 \times 0.963 \times 1.024)^2 + (0.003 \times 1.8464E-4 \times 1.73E6 \times 1.024)^2 + (0.002 \times 1.736E6 \times 0.963 \times 1.84635E-4)^2}$].

In the PWR-case $dt/dT = 2.0255E-04^{\circ}C/s$ is measured for 5A3. According to the equation above the total power is calculated to $P = 237.1 \text{ W}$, $P = (-86.36 + 2.0255E-4 \times 1.749E6 \times 0.915)(1+0.024)$.

The standard deviation is calculated to $\pm 4.6 \text{ W}$ [$s_p = \sqrt{(3.5 \times 1.024)^2 + (6,442 \times 2.0255E-04 \times 0.915 \times 1.024)^2 + (0.007 \times 2.0255E-4 \times 1.72E6 \times 1.024)^2 + (0.002 \times 0.915 \times 1.72E6 \times 2.0255E-4)^2}$].

An assessment of the uncertainty of the calorimeter over the whole measurement range could be done the same way.

BWR-assemblies are measured in the range 50– 350W. The standard deviation is calculated to ± 2.1 at 50 W and $\pm 3.1 \text{ W}$ at 350 W.

PWR-assemblies are measured in the 250– 900W. The standard deviation is calculated to $\pm 4.6 \text{ W}$ at 250 W and $\pm 9.4 \text{ W}$ at 900 W.

6 Conclusion

A procedure to calculate the standard deviation (s_p) in measured decay heat values has been developed. The resulting values for the calorimeter are:

BWR assembly 50 W: $s_p = 2.1$ W

BWR assembly 350 W: $s_p = 3.1$ W

PWR assembly 250 W: $s_p = 4.6$ W

PWR assembly 900 W: $s_p = 9.4$ W

The design target was that the uncertainty in the measured decay heat power should be less than $\pm 2\%$ with 95% confidence. 95% corresponds to $\pm 1.96s_p$

For BWR assemblies the uncertainty will be:

Power (W)	Uncertainty (W)	Uncertainty (%)
50	4.2	8.4
350	6.2	1.8

For PWR assemblies the uncertainty will be:

Power (W)	Uncertainty (W)	Uncertainty (%)
250	9.2	3.7
900	18.8	2.1

References

- /1/ Introductory Statistics, T. H: Wannacott et al.
- /2/ Uncertainty analysis, a tutorial, J Engel et al.
- /3/ Systembeskrivning Clab System 251.
- /4/ Variationer av temperaturen i höjdlid, OKG, F Sturek.
- /5/ Clab system 251 Volymbestämning av mätcyllindern, 2002-09-06, OKG, F Sturek.

Enclosure 1

Anpassning till 50–350 W

						a=	176.14
						b=	1,729,649.32
	X=dt/dT	Y=P	x=X-Xmedel	xY	x^2	y=a+bx	(Y-y)^2
	8.11E-05	49.1	-7.27E-05	-3.57E-03	5.29E-09	50.34	1.58
	8.18E-05	52.2	-7.21E-05	-3.76E-03	5.19E-09	51.49	0.45
	8.40E-05	54.6	-6.99E-05	-3.82E-03	4.88E-09	55.30	0.47
	1.08E-04	97.4	-4.58E-05	-4.46E-03	2.10E-09	96.91	0.29
	1.09E-04	97.7	-4.51E-05	-4.41E-03	2.03E-09	98.15	0.18
	1.08E-04	98.2	-4.54E-05	-4.46E-03	2.07E-09	97.53	0.40
	1.08E-04	98.2	-4.57E-05	-4.49E-03	2.09E-09	97.04	1.26
	1.09E-04	98.6	-4.51E-05	-4.45E-03	2.04E-09	98.06	0.27
	1.34E-04	143.4	-2.01E-05	-2.88E-03	4.02E-10	141.45	3.92
	1.36E-04	144.9	-1.76E-05	-2.54E-03	3.08E-10	145.78	0.69
	1.37E-04	146.3	-1.71E-05	-2.51E-03	2.94E-10	146.48	0.03
	1.38E-04	146.6	-1.63E-05	-2.39E-03	2.65E-10	147.97	1.76
	1.37E-04	147.1	-1.69E-05	-2.48E-03	2.85E-10	146.94	0.02
	1.35E-04	147.2	-1.85E-05	-2.72E-03	3.41E-10	144.19	9.33
	1.35E-04	147.3	-1.89E-05	-2.79E-03	3.58E-10	143.42	14.77
	1.38E-04	147.3	-1.55E-05	-2.28E-03	2.40E-10	149.35	4.34
	1.39E-04	147.3	-1.51E-05	-2.23E-03	2.29E-10	149.99	7.26
	1.38E-04	148.7	-1.59E-05	-2.36E-03	2.52E-10	148.69	0.00
	1.65E-04	192.9	1.12E-05	2.15E-03	1.25E-10	195.45	6.59
	1.65E-04	194.9	1.10E-05	2.14E-03	1.21E-10	195.15	0.05
	1.67E-04	196.9	1.27E-05	2.50E-03	1.62E-10	198.14	1.64
	1.69E-04	202.1	1.51E-05	3.05E-03	2.28E-10	202.28	0.03
	1.69E-04	202.2	1.51E-05	3.05E-03	2.28E-10	202.27	0.01
	1.94E-04	245.5	4.02E-05	9.87E-03	1.62E-09	245.70	0.05
	1.94E-04	245.9	4.03E-05	9.91E-03	1.62E-09	245.83	0.00
	1.93E-04	247.1	3.93E-05	9.72E-03	1.55E-09	244.16	8.66
	2.25E-04	294.6	7.13E-05	2.10E-02	5.08E-09	299.44	23.86
	2.25E-04	296.8	7.06E-05	2.10E-02	4.99E-09	298.33	2.28
	2.46E-04	336.8	9.25E-05	3.11E-02	8.55E-09	336.05	0.54
	2.49E-04	343.5	9.52E-05	3.27E-02	9.06E-09	340.81	7.23
	2.53E-04	349.1	9.91E-05	3.46E-02	9.83E-09	347.62	2.32
Summa			7.32E-19	1.24E-01	7.18E-08		100.27
Medel	1.54E-04	176.14				s^2 i a	3.46
						s i a	1.86
						s^2 i b	4,8136,343.10
						s i b	6,938.04

Anpassning till 50–350 W

a= 503.49
b= 1,716,827.82

	X=dt/dT	Y=P	x=X-Xmedel	xY	x^2	y=a+bx	(Y-y)^2
	1.94E-04	2.45E+02	-1.50E-04	-3.67E-02	2.24E-08	246.82	1.83
	1.94E-04	2.46E+02	-1.49E-04	-3.67E-02	2.23E-08	246.95	1.18
	1.93E-04	2.47E+02	-1.50E-04	-3.72E-02	2.26E-08	245.29	3.27
	2.25E-04	2.95E+02	-1.18E-04	-3.49E-02	1.40E-08	300.17	31.48
	2.25E-04	2.97E+02	-1.19E-04	-3.53E-02	1.42E-08	299.07	5.03
	2.46E-04	3.37E+02	-9.73E-05	-3.28E-02	9.46E-09	336.50	0.08
	2.49E-04	3.43E+02	-9.45E-05	-3.25E-02	8.93E-09	341.23	5.16
	2.53E-04	3.49E+02	-9.06E-05	-3.16E-02	8.20E-09	347.99	1.33
	2.77E-04	3.92E+02	-6.64E-05	-2.60E-02	4.40E-09	389.58	5.72
	2.80E-04	3.95E+02	-6.33E-05	-2.50E-02	4.01E-09	394.74	0.18
	3.07E-04	4.41E+02	-3.66E-05	-1.61E-02	1.34E-09	440.67	0.02
	3.08E-04	4.45E+02	-3.59E-05	-1.60E-02	1.29E-09	441.87	8.16
	3.36E-04	4.91E+02	-7.97E-06	-3.91E-03	6.35E-11	489.81	1.31
	3.41E-04	4.93E+02	-2.92E-06	-1.44E-03	8.54E-12	498.48	26.68
	3.38E-04	4.93E+02	-5.66E-06	-2.79E-03	3.21E-11	493.77	0.09
	3.61E-04	5.35E+02	1.70E-05	9.07E-03	2.87E-10	532.60	6.09
	3.64E-04	5.39E+02	2.00E-05	1.08E-02	3.98E-10	537.76	1.36
	3.67E-04	5.45E+02	2.38E-05	1.30E-02	5.67E-10	544.37	0.37
	3.93E-04	5.88E+02	4.95E-05	2.91E-02	2.45E-09	588.50	0.25
	3.92E-04	5.89E+02	4.83E-05	2.85E-02	2.33E-09	586.37	9.42
	3.96E-04	5.97E+02	5.24E-05	3.13E-02	2.75E-09	593.52	15.64
	4.48E-04	6.87E+02	1.05E-04	7.20E-02	1.10E-08	683.32	15.86
	4.56E-04	6.87E+02	1.12E-04	7.69E-02	1.25E-08	695.69	69.47
	4.49E-04	6.89E+02	1.05E-04	7.26E-02	1.11E-08	684.40	18.49
	4.78E-04	7.31E+02	1.35E-04	9.86E-02	1.82E-08	734.99	12.82
	4.83E-04	7.35E+02	1.39E-04	1.03E-01	1.94E-08	742.90	58.61
	5.05E-04	7.81E+02	1.61E-04	1.26E-01	2.60E-08	780.32	0.25
	5.63E-04	8.84E+02	2.19E-04	1.94E-01	4.81E-08	880.19	11.05
Summa			-3.87E-20	4.95E-01	2.88E-07		311.22
Medel	3.44E-04	503.49				s^2 i a	12.45
						s i a	3.53
						s^2 i b	43,163,868.01
						s i b	6569.92

Simplification of the formula for P_γ

According to part 3 is the formula for P_γ :

$$P_\gamma = 8.14 \frac{\sum_{i=1}^5 d_i e^{-\lambda r_i}}{\sum_{i=1}^5 (e^{-\lambda r_i})^2} \left\{ e^{-\lambda} \left(-\frac{1}{\lambda^2} - \frac{1}{\lambda} \right) + e^{-0.228\lambda} \left(\frac{1}{\lambda^2} + \frac{0.228}{\lambda} \right) \right\} F$$

Where

P_γ is the power escaping from the calorimeter with the gamma radiation (W)

$\lambda = 8.4 \text{ m}^{-1}$ is a curve fitting parameter to measured values of dose rates

F is a weigh factor compensating variations in the burnup profiles in the four corners of the fuel assembly

d_i is measured dose rate (Gy/h)

r_i is the distance between the probe and the fuel centre:

Prob nr (i)	Radie (m) (r_i)
1	0.26
2	0.38
3	0.50
4	0.63
5	0.77

By input of numerical values the formula could be simplified:

$$P_\gamma = 0.378d_{KA701} + 0.138d_{KA702} + 0.050d_{KA703} + 0.017d_{KA704} + 0.005d_{KA705}$$

Clab – Data for fuel assemblies used in calorimetric and nuclear measurements

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Contents

1	Introduction	167
2	Fuel data	169
2.1	Forsmark 1	171
2.2	Forsmark 2	183
2.3	Forsmark 3	193
2.4	Oskarshamn 2	200
2.5	Oskarshamn 3	215
2.6	Ringhals 1	221
2.7	Ringhals 2	264
2.8	Ringhals 3	271
2.9	Barsebäck 1	278
2.10	Barsebäck 2	293

1 Introduction

SKB is conducting measurements of decay heat and gamma radiation on spent fuel assemblies in Clab.

This report describes the fuel data on the 50 BWR – and 34 PWR – assemblies in the present measurement program. The fuel assemblies are from different nuclear power plants:

Forsmark 1

Forsmark 2

Forsmark 3

Oskarshamn 2

Oskarshamn 3

Ringhals 1

Ringhals 2

Ringhals 3

Barsebäck 1

Basebäck 2

In Section 2 data is presented for each reactor and the fuel assemblies from the reactors. Fuel rod diagrams and enrichment diagrams are presented for the fuel assemblies. These diagrams are copies of the plant original data. The diagrams are not translated and we regret that the quality of some diagrams is not good, but all are included for completeness. In Table 2-1 below all fuel assemblies are listed by id number, fuel type and reactor.

The data has been collected from the power plants.

2 Fuel data

Table 2-1. Assemblies listed by number.

BWR-assemblies			PWR-assemblies		
Assembly no	Fuel type	Reactor	Assembly no	Fuel type	Reactor
0582	8x8	R1	0E2	17x17	R3
0596	8x8	R1	0E6	17x17	R3
0710	8x8	R1	1E5	17x17	R3
0900	8x8	R1	0C9	17x17	R3
1136	8x8	R1	1C2	17x17	R3
1177	8x8	R1	1C5	17x17	R3
1186	8x8	R1	2A5	17x17	R3
1377	8x8	O2	2C2	17x17	R3
1389	8x8	O2	3C1	17x17	R3
1546	8x8	O2	3C4	17x17	R3
1696	8x8	O2	3C5	17x17	R3
1704	8x8	O2	3C9	17x17	R3
2014	8x8	B1	4C4	17x17	R3
2018	8x8	B1	4C7	17x17	R3
2048	8x8	B1	5A3	17x17	R3
2074	8x8	B1	5F2	17x17	R3
2118	8x8	B1	C01	15x15	R2
2995	8x8	O2	C12	15x15	R2
3054	8x8	O2	C20	15x15	R2
3058	8x8	O2	C42	15x15	R2
3064	8x8	O2	D27	15x15	R2
3838	8x8	F1	D38	15x15	R2
5535	8x8	F2	E38	15x15	R2
5829	8x8	R1	E40	15x15	R2
6350	8x8	O2	F14	15x15	R2
6423	8x8	R1	F21	15x15	R2
6432	8x8	R1	F25	15x15	R2
6454	8x8	R1	F32	15x15	R2
6478	8x8	R1	G11	15x15	R2
8327	8x8	R1	G23	15x15	R2
8331	8x8	R1	I09	15x15	R2
8332	8x8	R1	I20	15x15	R2
8338	8x8	R1	I24	15x15	R2
8341	8x8	R1	I25	15x15	R2
9329	8x8	B1			
288	8x8	B1			

BWR-assemblies			PWR-assemblies		
Assembly no	Fuel type	Reactor	Assembly no	Fuel type	Reactor
11494	Svea 64	F2			
11495	Svea 64	F2			
12078	8x8	O3			
12684	Svea 64	O2			
13628	Svea 100	O3			
13630	Svea 100	O3			
13775	Svea 64	F2			
13847	Svea 100	F3			
13848	Svea 100	F3			
14076	8x8	B2			
KU0100	8x8-2	F1			
KU0269	9x9-5	F1			
KU0278	9x9-5	F1			
KU0282	9x9-5	F1			

2.1 Forsmark 1

REACTOR FORSMARK 1

Reactor data	Value
Reactor pressure (MPa) at full power	7
Inlet temperature into core (C) at full power	274
Outlet temperature into core (C) at full power	286
Avg. temperature in the uranium pellets at full power (C)	508
Avg. temperature in the cladding at full power (C)	290
Temperature in the boxwall at full power (C)	286

Cycle data

Cycle no	Start	Stop	Reactor power MW	Burnup avg/cycle MWd/tU	No of assemblies in core
1a	1980-04-23	1981-06-25	2,711	5,894	676
1b	1981-08-16	1982-06-19	2,711	6,156	676
2	1982-08-25	1983-07-15	2,711	6,156	676
3	1983-08-05	1984-07-13	2,711	6,375	676
4	1984-08-06	1985-05-31	2,711	5,848	676
5	1985-07-08	1986-07-04	2,928	6,959	676
6	1986-07-22	1987-07-31	2,928	7,770	676
7	1987-08-19	1988-06-10	2,928	6,404	676
8	1988-07-10	1989-07-14	2,928	7,253	676
9	1989-08-18	1990-08-17	2,928	7,129	676
10	1990-09-16	1991-05-24	2,928	5,686	676
11	1991-06-20	1992-07-10	2,928	8,347	676
12	1992-08-03	1993-07-02	2,928	7,092	676

Fuel data

	Fuel id				
	3838	KU0100	KU0269	KU0278	KU0282
Fuel type	AA8x8-1	KWU8x8-2	KWU9x9-5	KWU9x9-5	KWU9x9-5
No of fuel rods	63	62	76	76	76
Rod pitch normal rods (mm)	16.3	16.25	14.45	14.45	14.45
Rod pitch normal – corner rods (mm)	16.05				
Rod pitch corner – corner rods (mm)	15.8				
Rod diameter normal rod (mm)	12.25	12.3	11	11	11
Clad thickness normal rod (mm)	0.8	0.82	0.665	0.665	0.665
Pellet diameter normal rod (mm)	10.44	10.44	9.5	9.5	9.5
No of normal rods	51	62	76	76	76
Rod diameter corner rod (mm)	11.75				
Clad thickness corner rod (mm)	0.8				
Pellet diameter corner rod (mm)	9.94				
No of corner rods	12				
Active length (mm)	3,680	3,680	3,680	3,680	3,680
Density UO ₂ (g/cc)	10.5	10.3	10.3	10.3	10.3
Porsity,dishing etc (%)	1.29	1.255	1.545	1.483	1.504
No of spacer rods	1	1	1	1	1
Material in spacer rods	Zr2	Zr	Zr	Zr	Zr
Outer diameter spacer rods (mm)	12.25	15	15	15	15
Cladding thickness spacer rod	0.8	0.8	0.8	0.8	0.8
No of water rods		1	4	4	4
Material in water rods		Zr2	Zr2	Zr2	Zr2
Outer diameter water rods (mm)		15	15	15	15
Inner diameter water rods (mm)		0.8	0.8	0.8	0.8
No of BA rods	3	4	6	6	6
% Gd ₂ O ₃	3.95	2.5	2.5	2.5	2.5
Rel poison	1	1	1	1	1
No of spacers	6	6	6	6	6
Spacer material	Inconel	Zr4	Zr4	Zr4	Zr4
Mass of spacer (g)	135	323	323	323	323
Box material	Zr4	Zr4	Zr4	Zr4	Zr4
Box outer measure square (mm)	139	139	139	139	139
Box wall thickness (mm)	2.3	2.3	2.3	2.3	2.3

Initial data	3838	KU0100	KU0269	KU0278	KU0282
Initial mass Utot (g)	177,903	174,920	177,020	177,133	177,097
Initial mass U235 (g)	3,711	5,190	5,201	5,206	5,204
Avg.enrichment% U235	2.086	2.976	2.938	2.939	2.939
Data after rebuild 1					
Date of rebuild					
No of fuel rods					
No spacer rods					
No water rods					
No of water holes					
Mass Utot after rebuild (g)					
Mass U235 after rebuild (g)					
Data after rebuild 2					
Date of rebuild					
No of fuel rods					
No spacer rods					
No water rods					
No of water holes					
Mass Utot after rebuild (g)					
Mass U235 after rebuild (g)					
Cycle history burnup/cycle, MWd/tU					
1a	6,160				
1b	6,932				
2	2,898				
3	3,063				
4					
5					
6		10,643	10,158	10,414	10,122
7		8,063	8,629	7,271	8,678
8		7,924	8,366	7,079	7,044
9		7,563	7,960	4,649	6,268
10	2,786			5,910	5,784
11	3,830				
Axial BU distribution, EOL MWd/tU Node no (top=25)					
1	10,899	14,315	7,452	6,964	8,156
2	19,258	24,831	27,664	25,831	29,042
3	23,070	30,845	34,966	32,613	36,483
4	24,864	33,763	38,158	35,857	39,916
5	26,611	35,914	40,116	38,143	42,260
6	27,703	36,991	40,833	39,258	43,308
7	28,282	37,557	40,928	39,764	43,593
5	26,611	35,914	40,116	38,143	42,260
6	27,703	36,991	40,833	39,258	43,308
7	28,282	37,557	40,928	39,764	43,593
8	28,538	37,745	40,613	39,739	43,278
9	29,705	38,515	41,035	40,345	43,629
10	30,180	38,848	41,198	40,592	43,659
11	30,167	39,001	41,264	40,697	43,665
12	29,904	38,735	40,984	40,415	43,257
13	30,323	39,103	41,326	40,772	43,514
14	30,356	39,150	41,205	40,759	43,405
15	30,116	38,986	40,782	40,520	43,055
16	29,719	38,482	39,990	40,005	42,397
17	29,633	38,550	39,746	40,153	42,397
18	29,031	38,243	39,094	40,085	42,114
19	27,866	37,617	38,068	39,785	41,579
20	26,758	36,474	36,449	38,917	40,466
21	25,237	35,470	34,847	38,102	39,470
22	22,794	33,379	32,126	35,972	37,256
23	19,043	29,888	27,929	31,934	33,273
24	14,381	24,090	22,053	25,653	26,987
25	10,623	18,330	9,013	10,199	11,231

Void history	3838	KU0100	KU0269	KU0278	KU0282
Node no (top=25)					
1	-0.04	-0.03	-0.04	-0.03	-0.03
2	-0.03	-0.03	-0.03	-0.03	-0.03
3	-0.03	-0.02	-0.01	-0.02	-0.02
4	0.00	0.02	0.05	0.02	0.04
5	0.03	0.09	0.13	0.08	0.11
6	0.06	0.16	0.21	0.15	0.17
7	0.10	0.24	0.28	0.21	0.24
8	0.13	0.30	0.35	0.27	0.30
9	0.17	0.36	0.41	0.32	0.35
10	0.22	0.42	0.46	0.37	0.41
11	0.27	0.46	0.51	0.42	0.45
12	0.31	0.51	0.55	0.46	0.49
13	0.35	0.54	0.58	0.50	0.53
14	0.39	0.58	0.62	0.54	0.56
15	0.42	0.61	0.64	0.57	0.59
16	0.45	0.63	0.67	0.60	0.62
17	0.48	0.66	0.69	0.62	0.65
18	0.50	0.68	0.71	0.65	0.67
19	0.53	0.70	0.73	0.66	0.68
20	0.54	0.71	0.74	0.68	0.70
21	0.56	0.73	0.75	0.70	0.71
22	0.57	0.74	0.76	0.71	0.73
23	0.59	0.75	0.77	0.72	0.73
24	0.59	0.76	0.78	0.73	0.74
25	0.61	0.76	0.78	0.73	0.74
Density history					
Node no (top=25)					
2	0.76	0.76	0.76	0.76	0.76
3	0.75	0.75	0.74	0.75	0.74
4	0.74	0.72	0.70	0.72	0.71
5	0.72	0.68	0.65	0.68	0.66
6	0.69	0.63	0.59	0.64	0.62
7	0.67	0.57	0.54	0.60	0.57
8	0.64	0.53	0.50	0.56	0.54
9	0.61	0.48	0.46	0.52	0.50
10	0.58	0.44	0.42	0.49	0.46
11	0.55	0.41	0.39	0.46	0.43
12	0.52	0.38	0.36	0.43	0.40
13	0.49	0.36	0.34	0.40	0.38
14	0.46	0.33	0.32	0.37	0.35
15	0.44	0.31	0.30	0.35	0.33
16	0.42	0.30	0.28	0.33	0.31
17	0.40	0.28	0.27	0.31	0.30
18	0.38	0.27	0.25	0.30	0.28
19	0.36	0.25	0.24	0.29	0.27
20	0.35	0.24	0.23	0.27	0.26
21	0.34	0.23	0.22	0.26	0.25
22	0.33	0.22	0.21	0.25	0.24
23	0.32	0.22	0.21	0.25	0.24
24	0.32	0.21	0.20	0.24	0.23
25	0.31	0.21	0.20	0.24	0.23

Control rod history
Node no (top=25)

1	0.08	0.00	0.12	0.22	0.28
2	0.06	0.00	0.10	0.17	0.24
3	0.05	0.00	0.10	0.15	0.23
4	0.04	0.00	0.10	0.15	0.23
5	0.04	0.00	0.10	0.15	0.22
6	0.04	0.00	0.09	0.14	0.22
7	0.04	0.00	0.08	0.14	0.23
8	0.03	0.00	0.08	0.14	0.23
9	0.01	0.00	0.06	0.14	0.23
10	0.01	0.00	0.05	0.13	0.23
11	0.01	0.00	0.04	0.12	0.22
12	0.01	0.00	0.03	0.11	0.21
13	0.00	0.00	0.01	0.10	0.20
14	0.00	0.00	0.00	0.09	0.20
15	0.00	0.00	0.00	0.09	0.19
16	0.00	0.00	0.00	0.08	0.19
17	0.00	0.00	0.00	0.08	0.18
18	0.00	0.00	0.00	0.08	0.18
19	0.00	0.00	0.00	0.07	0.16
20	0.00	0.00	0.00	0.05	0.15
21	0.00	0.00	0.00	0.03	0.12
22	0.00	0.00	0.00	0.01	0.10
23	0.00	0.00	0.00	0.00	0.06
24	0.00	0.00	0.00	0.00	0.02
25	0.00	0.00	0.00	0.00	0.00

Forsmark 1 Fuel rod diagram assembly 3838



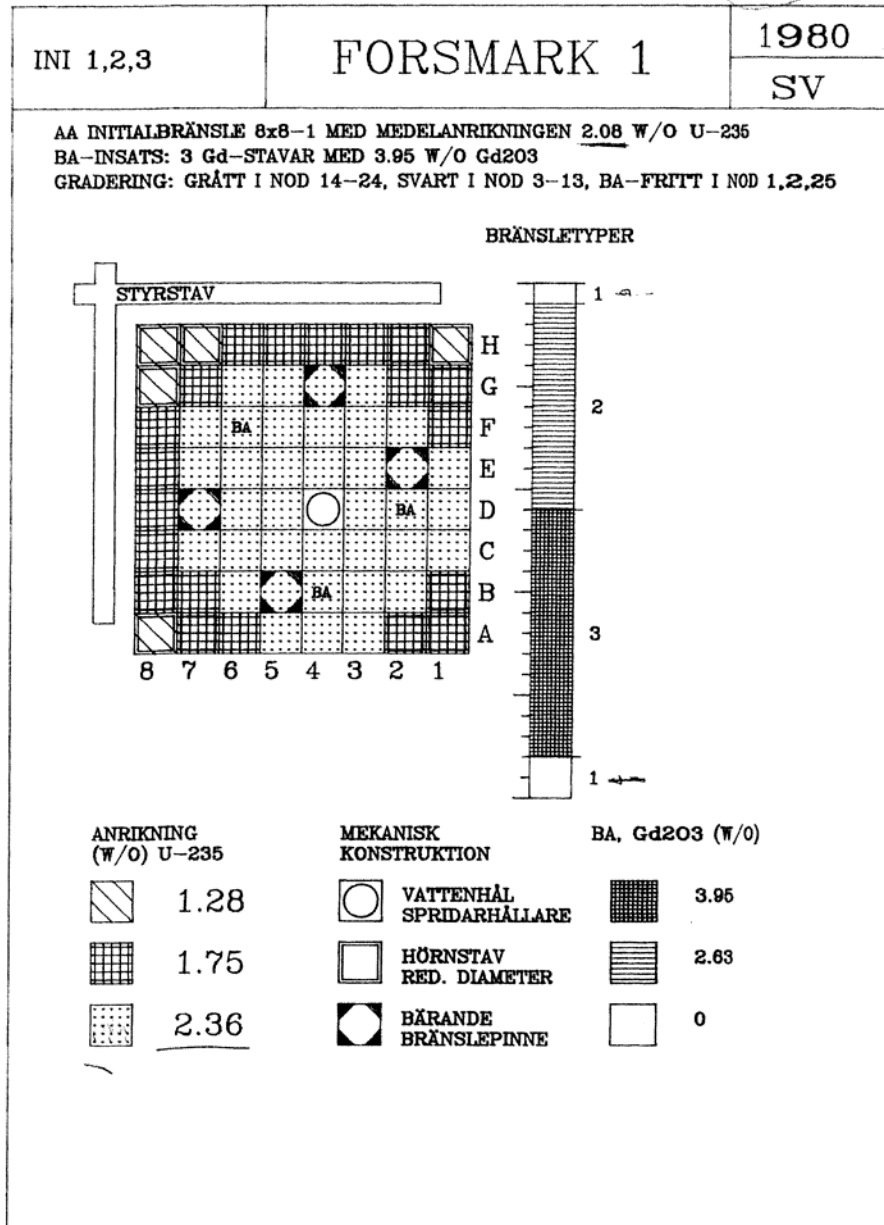
PATRON NR 3838
BOX NR 3916

ANRIKN %	ANTAL	KG	KG	KG
VERKL	STAVAR	UO2	UTOT	U235
2.086	63	202.220	177.903	3.711

		000-	000-	000-	000-	000-	000-	000-
		9509	11359	36416	36403	36408	36380	9536
8		1.271	1.729	1.749	1.748	1.747	1.751	1.307
		2596	2603	2858	2861	2862	2856	2601
		000-	000-	000-	000-	000-	000-	000-
		11307	36308	30293	2697	30335	30308	36404
7		1.734	1.747	2.349	2.359	2.351	2.348	1.746
		2595	2862	2895	2883	2893	2896	2864
		000-	000-	000-	000-	000-	000-	000-
		36423	30282	30180	30304	30311	16511	30344
6		1.742	2.346	2.355	2.346	2.352	2.346	2.353
		2871	2898	2888	2898	2891	2771	2890
		000-	000-	000-	000-	000-	000-	000-
		30289	2635	30341	30322	30147	30260	30300
5		2.351	2.366	2.352	2.355	2.351	2.351	2.353
		2893	2874	2891	2888	2893	2893	2890
		000-	000-	000-		000-	000-	000-
		30251	16506	30285		30135	30301	2680
4		2.347	2.341	2.352		2.356	2.352	2.358
		2897	2777	2891		2886	2891	2884
		000-	000-	000-	000-	000-	000-	000-
		30271	30324	30170	30316	30336	30345	30270
3		2.352	2.358	2.358	2.349	2.350	2.353	2.350
		2891	2884	2884	2895	2894	2890	2894
		000-	000-	000-	000-	000-	000-	000-
		11314	30318	30334	16527	2591	30346	36410
2		1.732	2.352	2.352	2.347	2.369	2.344	1.743
		2598	2891	2891	2769	2870	2901	2869
		000-	000-	000-	000-	000-	000-	000-
		11474	11306	30283	30288	30339	36338	11524
1		1.760	1.732	2.346	2.349	2.350	1.755	1.762
		2613	2598	2898	2895	2894	2849	2610
		A	B	C	D	E	F	G
								H

3838

2.5



Forsmark 1 Fuel rod diagram assembly KU0100



PATRON NR KU0100
BOX NR 3592

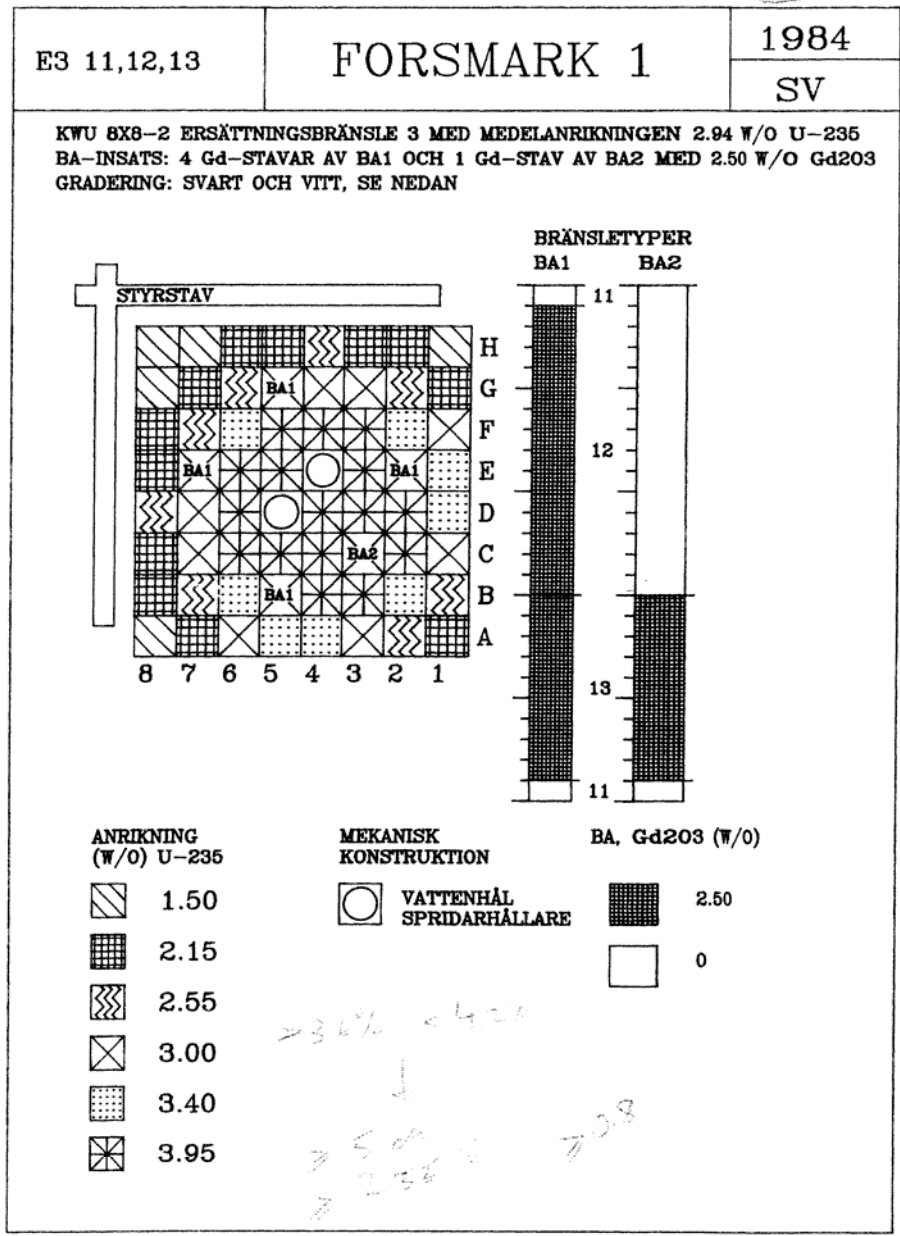
ANRIKN %	ANTAL	KG	KG	KG
VERKL	STAVAR	UO2	UTOT	U235
2.967	62	198.760	174.920	5.190

	K02-	K02-	K02-	K02-	K02-	K02-	K02-	K02-
8	AM0290	AN0288	AN0302	AP0202	AO0035	AN0295	AM0377	AL0118
	1.00	2.00	2.00	2.00	2.00	2.00	1.00	1.00
	3212	3204	3204	3207	3207	3209	3219	3217
7	K02-	K02-	K02-	K02-	K02-	K02-	K02-	K02-
	AN0550	AQ0075	AD0229	AD0248	AR0102	AQ0067	AN0310	AM0262
	2.00	3.00	3.00	3.00	3.00	2.00	2.00	2.00
	3195	3203	3205	3199	3180	3202	3194	3224
6	K02-	K02-	K02-	K02-	K02-	K02-	K02-	K02-
	AD0241	AS0290	AT1058	AT1141	AT1033	AS0412	AQ0103	AN1042
	3.00	3.00	3.00	3.00	3.00	3.00	2.00	2.00
	3197	3217	3204	3208	3205	3221	3205	3215
5	K02-	K02-	K02-		K02-	K02-	K02-	K02-
	AU0087	AR0114	AT1412		AT1075	AT1377	AR0058	AQ0187
	3.00	3.00	3.00		3.00	3.00	3.00	2.00
	3179	3167	3210		3211	3211	3178	3215
4	K02-	K02-	K02-	K02-		K02-	K02-	K02-
	AU0013	AT1022	AT1430	AT1011		AT1410	AD0475	AP0233
	3.00	3.00	3.00	3.00		3.00	3.00	2.00
	3184	3213	3211	3216		3215	3191	3218
3	K02-	K02-	K02-	K02-	K02-	K02-	K02-	K02-
	AD0236	AT1096	AV0051	AT1052	AT1104	AT1073	AD0463	AN1046
	3.00	3.00	3.00	3.00	3.00	3.00	3.00	2.00
	3206	3216	3186	3216	3207	3207	3188	3217
2	K02-	K02-	K02-	K02-	K02-	K02-	K02-	K02-
	AQ0495	AS0381	AT1013	AT1134	AR0403	AS0376	AQ0079	AN1041
	2.00	3.00	3.00	3.00	3.00	3.00	2.00	2.00
	3208	3218	3215	3215	3184	3224	3210	3210
1	K02-	K02-	K02-	K02-	K02-	K02-	K02-	K02-
	AN0290	AQ0065	AD0243	AU0056	AU0027	AD0454	AN0303	AM0298
	2.00	3.00	3.00	3.00	3.00	3.00	2.00	1.00
	3205	3199	3201	3221	3212	3205	3201	3217
	A	B	C	D	E	F	G	H

Forsmark 1 Enrichment diagram assembly KU0100

KU0100

6.72



Forsmark 1 Fuel rod diagram assembly KU0269



PATRON NR KU0269
BOX NR 3896

ANRIKN %	ANTAL	KG	KG	KG
VERKL	STAVAR	UO2	UTOT	U235
2.938	76	201.203	177.020	5.201

	K05-	K05-	K05-	K05-	K05-	K05-	K05-	K05-	K05-
9	BR0007	BR0330	BS0729	BQ0263	BS0736	BQ0250	BR0601	BR0297	BP0044
	2.00	2.00	2.60	2.60	2.60	2.60	2.00	2.00	1.50
	2665	2665	2674	2670	2667	2676	2659	2667	2668
8	K05-	K05-	K05-	K05-	K05-	K05-	K05-	K05-	K05-
	BS0630	BL0056	BN0138	BT0496	BU1568	BL0053	BN0140	BS0295	BR0276
	2.60	3.00	3.00	3.70	3.95	3.00	3.00	2.60	2.00
	2675	2639	2665	2665	2678	2642	2671	2671	2666
7	K05-	K05-	K05-	K05-	K05-	K05-	K05-	K05-	K05-
	BN0983	BT0380	BU1580	BU1599	BU1570	BU1518	BN0985	BN0261	BR0408
	3.00	3.70	3.95	3.95	3.95	3.95	3.00	3.00	2.00
	2670	2679	2669	2667	2675	2673	2670	2659	2667
6	K05-	K05-	K05-	K05-	K05-	K05-	K05-	K05-	K05-
	BH0183	BU1566	BU1476	BU1609	BU1436	BU1469	BU1553	BL0037	BQ0192
	3.00	3.95	3.95	3.95	3.95	3.95	3.95	3.00	2.60
	2664	2674	2669	2673	2668	2667	2672	2632	2680
5	K05-	K05-	K05-		K05-	K05-	K05-	K05-	K05-
	BN0966	BL0492	BT0151		BT0470	BU1542	BU1539	BU1504	BS0601
	3.00	3.00	3.70		3.70	3.95	3.95	3.95	2.60
	2670	2639	2674		2671	2672	2668	2676	2678
4	K05-	K05-				K05-	K05-	K05-	K05-
	BH0028	BT0419				BU1559	BU1560	BT0410	BQ0248
	3.00	3.70				3.95	3.95	3.70	2.60
	2662	2671				2666	2668	2672	2681
3	K05-	K05-	K05-		K05-	K05-	K05-	K05-	K05-
	BN0253	BT0479	EK0070		BT0474	BU1448	BU1417	BN0198	BS0510
	3.00	3.70	3.00		3.70	3.95	3.95	3.00	2.60
	2667	2676	1163		2666	2662	2672	2663	2673
2	K05-	K05-	K05-	K05-	K05-	K05-	K05-	K05-	K05-
	BS0743	BN0284	BT0506	BT0421	BL0276	BU1337	BT0485	BL0051	BR0230
	2.60	3.00	3.70	3.70	3.00	3.95	3.70	3.00	2.00
	2673	2667	2664	2674	2624	2673	2674	2648	2670
1	K05-	K05-	K05-	K05-	K05-	K05-	K05-	K05-	K05-
	BR0724	BS0640	BN0004	BH0084	BN0988	BH0198	BN0977	BS0605	BR0708
	2.00	2.60	3.00	3.00	3.00	3.00	3.00	2.60	2.00
	2677	2674	2669	2664	2662	2663	2669	2669	2668
	A	B	C	D	E	F	G	H	I

Forsmark 1 Fuel rod diagram assembly KU0278



PATRON NR KU0278
BOX NR 11B367

ANRIKN % VERKL	ANTAL STAVAR	KG UO2	KG UTOT	KG U235
2.939	76	201.335	177.133	5.206

	K05-	K05-	K05-	K05-	K05-	K05-	K05-	K05-	K05-
9	BR0671	BR0331	BS0746	BQ0343	BS0610	BQ0312	BR0410	BR0475	BP0087
	2.00	2.00	2.60	2.60	2.60	2.60	2.00	2.00	1.50
	2667	2663	2669	2669	2672	2666	2674	2675	2669
8	BS0612	BL0361	BN0904	BT0478	BU1180	BL0219	BN0110	BS0672	BR0578
	2.60	3.00	3.00	3.70	3.95	3.00	3.00	2.60	2.00
	2677	2648	2668	2673	2674	2647	2671	2680	2658
7	BN0090	BT0624	BU0946	BU1146	BU1112	BU1158	BN0960	BN0935	BR0220
	3.00	3.70	3.95	3.95	3.95	3.95	3.00	3.00	2.00
	2660	2692	2672	2675	2672	2673	2669	2662	2671
6	BH0076	BU1167	BU1182	BU1132	BU0903	BU1220	BU0898	BL0113	BQ0146
	3.00	3.95	3.95	3.95	3.95	3.95	3.95	3.00	2.60
	2669	2666	2667	2666	2671	2672	2670	2652	2669
5	BN0013	BL0194	BT0490		K05- BT0614	K05- BU1116	K05- BU1589	K05- BU1168	K05- BS0589
	3.00	3.00	3.70		3.70	3.95	3.95	3.95	2.60
	2665	2649	2671		2684	2671	2666	2671	2677
4	BH0221	K05- BT0688				K05- BU1236	K05- BU1129	K05- BT0685	K05- BQ0114
	3.00	3.70				3.95	3.95	3.70	2.60
	2663	2691				2669	2667	2683	2665
3	BN0196	K05- BT0248	K05- BK0017		K05- BT0430	K05- BU0968	K05- BU1593	K05- BN0170	K05- BS0681
	3.00	3.70	3.00		3.70	3.95	3.95	3.00	2.60
	2665	2670	1171		2673	2670	2668	2666	2678
2	BS0721	K05- BN0081	K05- BT0047	K05- BT0345	K05- BL0362	K05- BU1164	K05- BT0298	K05- BL0203	K05- BR0676
	2.60	3.00	3.70	3.70	3.00	3.95	3.70	3.00	2.00
	2672	2662	2679	2673	2650	2672	2680	2645	2673
1	BR0604	K05- BS0631	K05- BN0343	K05- BH0204	K05- BN0271	K05- BH0131	K05- BN0956	K05- BS0571	K05- BR0125
	2.00	2.60	3.00	3.00	3.00	3.00	3.00	2.60	2.00
	2673	2676	2668	2663	2669	2664	2662	2671	2662
	A	B	C	D	E	F	G	H	I

Forsmark 1 Fuel rod diagram assembly KU0282



PATRON NR KU0282
BOX NR 11B628

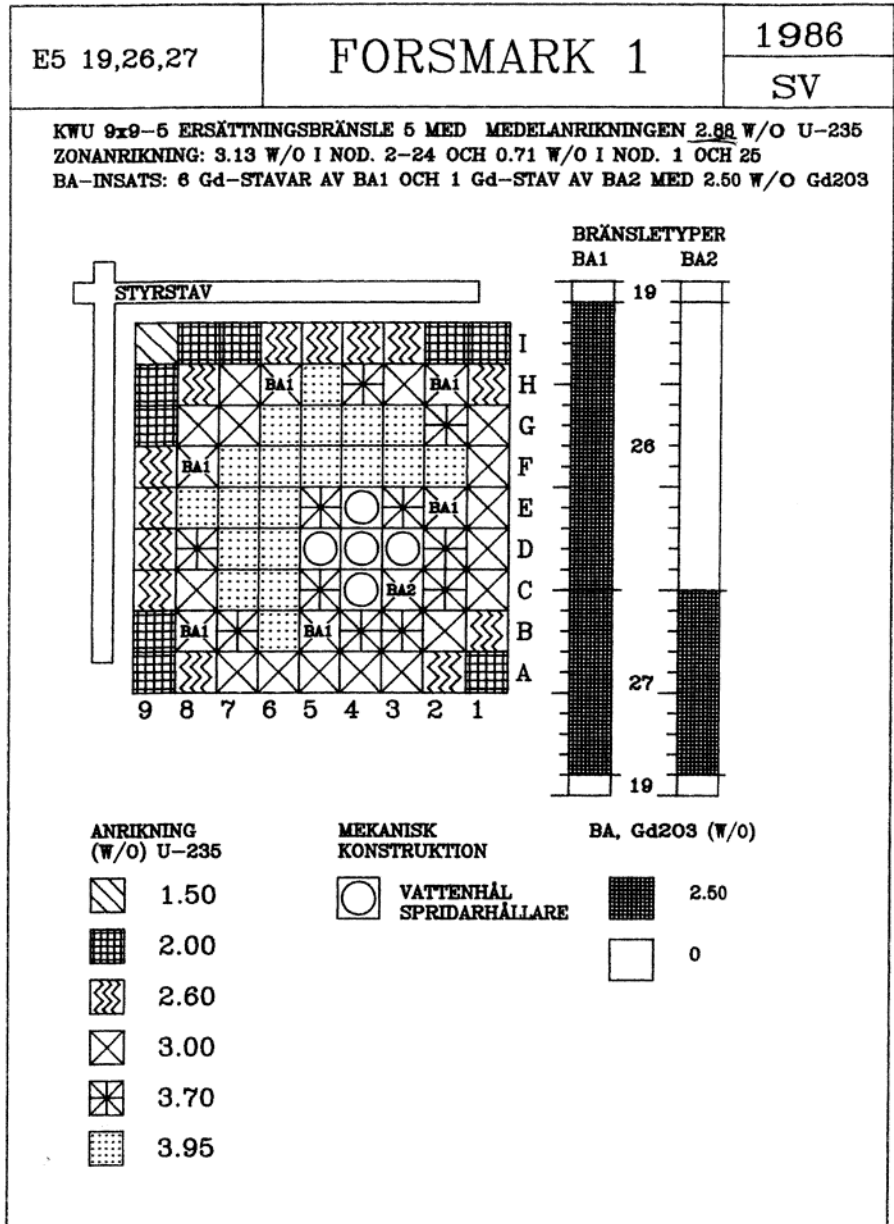
ANRIKN %	ANTAL	KG	KG	KG
VERKL	STAVAR	UO2	UTOT	U235
2.939	76	201.296	177.097	5.204

	K05-	K05-	K05-	K05-	K05-	K05-	K05-	K05-	K05-
9	BR0055	BR0034	BS0748	BQ0306	BS0719	BQ0266	BR0462	BR0391	BP0003
	2.00	2.00	2.60	2.60	2.60	2.60	2.00	2.00	1.50
	2676	2670	2675	2669	2671	2661	2673	2664	2669
8	BS0544	BL0114	BN0009	BT0130	BU0890	BL0101	BN0039	BS0713	BR0461
	2.60	3.00	3.00	3.70	3.95	3.00	3.00	2.60	2.00
	2671	2647	2663	2674	2670	2649	2662	2671	2669
7	BN0192	BT0205	BU0938	BU1202	BU1145	BU1243	BN0335	BN0056	BR0392
	3.00	3.70	3.95	3.95	3.95	3.95	3.00	3.00	2.00
	2664	2668	2672	2670	2671	2670	2667	2665	2668
6	BH0319	BU0930	BU0897	BU1207	BU1200	BU0948	BU1140	BL0163	BQ0162
	3.00	3.95	3.95	3.95	3.95	3.95	3.95	3.00	2.60
	2668	2674	2669	2671	2671	2671	2670	2658	2668
5	BN0064	BL0169	BT0246		BT0236	BU1187	BU1245	BU1099	BS0584
	3.00	3.00	3.70		3.70	3.95	3.95	3.95	2.60
	2666	2656	2672		2674	2673	2671	2667	2673
4	BH0123	BT0398				BU1089	BU1128	BT0256	BQ0239
	3.00	3.70				3.95	3.95	3.70	2.60
	2663	2677				2671	2675	2679	2663
3	BN0358	BT0441	BK0050		BT0311	BU1600	BU1135	BN0341	BS0415
	3.00	3.70	3.00		3.70	3.95	3.95	3.00	2.60
	2663	2672	1174		2672	2671	2671	2664	2677
2	BS0508	BN0238	BT0291	BT0188	BL0110	BU1218	BT0271	BL0108	BR0221
	2.60	3.00	3.70	3.70	3.00	3.95	3.70	3.00	2.00
	2674	2666	2670	2674	2658	2668	2673	2651	2669
1	BR0134	BS0506	BN0118	BH0305	BN0052	BH0247	BN0376	BS0423	BR0592
	2.00	2.60	3.00	3.00	3.00	3.00	3.00	2.60	2.00
	2670	2676	2665	2664	2669	2668	2662	2673	2663
	A	B	C	D	E	F	G	H	I

Forsmark 1 Enrichment diagram assembly KU0269, KU0278 and KU0282

KU0269
KU0278
KU0282

3.15



2.2 Forsmark 2

REACTOR FORSMARK 2

Reactor data	Value
Reactor pressure (MPa) at full power	7
Inlet temperature into core (C) at full power	273.7
Outlet temperature into core (C) at full power	286
Avg. temperature in the uranium pellets at full power (C)	486
Avg. temperature in the cladding at full power (C)	290
Temperature in the boxwall at full power (C)	286

Cycle data

Cycle no	Start	Stop	Reactor power MW	Burnup avg/cycle MWd/tU	No of assemblies in core
1a	1980-12-01	1982-05-01	2,711	7,214	676
1b	1982-08-18	1983-05-20	2,711	5,492	676
2	1983-06-11	1984-05-25	2,711	5,492	676
3	1984-07-13	1985-07-05	2,711	6,886	676
4	1985-07-26	1986-07-25	2,711	6,672	676
5	1986-08-09	1987-05-31	2,928	6,395	676
6	1987-06-20	1988-07-15	2,928	7,882	676
7	1988-08-04	1989-08-18	2,928	6,775	676
8	1989-09-04	1990-05-18	2,928	5,079	676
9	1990-06-12	1991-07-12	2,928	8,185	676
10	1991-07-31	1992-08-14	2,928	7,789	676
11	1992-08-31	1993-07-30	2,928	6,818	676
12	1993-09-07	1994-05-28	2,928	5,918	676

Fuel data	Fuel id 5535	11494	11495	13775
Fuel type	8x8-1	Svea 64	Svea 64	Svea 64
No of fuel rods	63	63	63	63
Rod pitch normal rods (mm)	16.3	15.8	15.8	15.8
Rod pitch normal – corner rods (mm)	16.05			
Rod pitch corner – corner rods (mm)	15.8			
Rod diameter normal rod (mm)	12.25	12.25	12.25	12.25
Clad thickness normal rod (mm)	0.8	0.8	0.8	0.8
Pellet diameter normal rod (mm)	10.44	10.44	10.44	10.44
No of normal rods	51	63	63	63
Rod diameter corner rod (mm)	11.75			
Clad thickness corner rod (mm)	0.8			
Pellet diameter corner rod (mm)	9.94			
No of corner rods	12			
Active length (mm)	3,680	3,680	3,680	3,680
Density UO ₂ (g/cc)	10.5	10.5	10.5	10.5
Porsity,dishing etc (%)	1.41	1.31	1.32	1.175
	10.35195	10.36245	10.3614	10.376625
No of spacer rods	1	1	1	1
Material in spacer rods	Zr2	Zr2	Zr2	Zr2
Outer diameter spacer rods (mm)	12.25	12.25	12.25	12.25
Cladding thickness spacer rod	0.8	0.8	0.8	0.8
No of water rods		1	4	4
Material in water rods		Zr2	Zr2	Zr2
Outer diameter water rods (mm)		15	15	15
Inner diameter water rods (mm)		0.8	0.8	0.8
No of BA rods		4	4	5
% Gd ₂ O ₃		2.55	2.55	2.55
Rel poison		0.82	0.82	1
No of spacers	6	4x6	4x6	4x6
Spacer material	Inconel	Inconel	Inconel	Inconel
Mass of spacer (g)	135	35	35	35
Box material	Zr4	Zr4	Zr4	Zr4
Box outer measure square (mm)	139	140	140	140
Box wall thickness (mm)	2.3	1.1	1.1	1.1

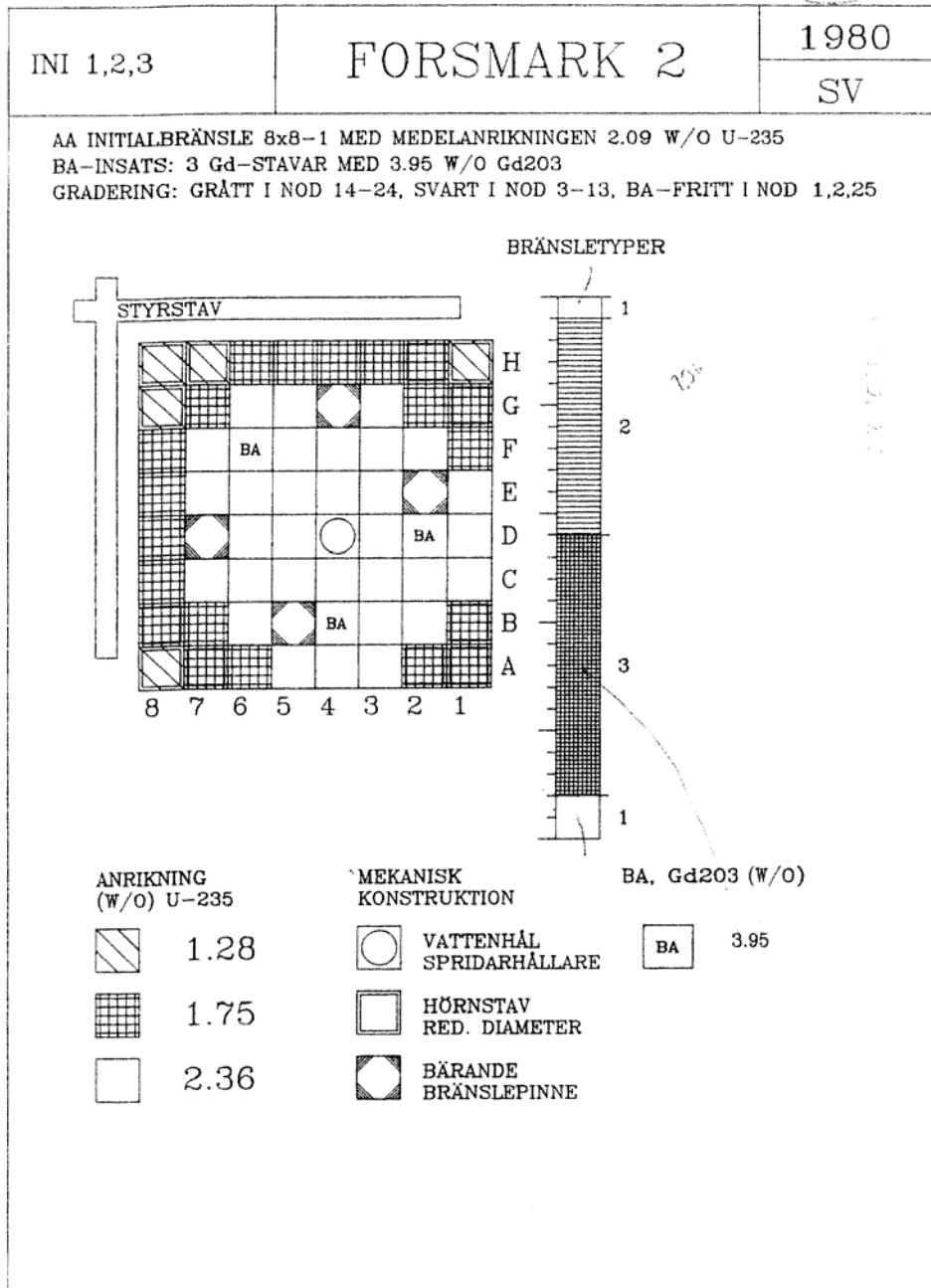
Initial data	5535	11494	11495	13775
Initial mass Utot (g)	177,689	181,088	181,070	181,340
Initial mass U235 (g)	3,723	5,287	5,287	5,175
Avg.enrichment% U235	2.095	2.920	2.910	2.850
Data after rebuild 1				
Date of rebuild				
No of fuel rods				
No spacer rods				
No water rods				
No of water holes				
Mass Utot after rebuild (g)				
Mass U235 after rebuild (g)				
Data after rebuild 2				
Date of rebuild				
No of fuel rods				
No spacer rods				
No water rods				
No of water holes				
Mass Utot after rebuild (g)				
Mass U235 after rebuild (g)				
Cycle history burnup/cycle, MWd/tU				
1a	8,850			
1b	1,908			
2	2,104			
3	2,120	9,098	9,098	
4	1,683	7,578	7,578	
5	1,450	7,460	7,460	8,801
6	1,829	8,295	8,295	9,499
7				5,675
8				3,523
9				5,339
10				
Axial BU distribution, EOL MWd/tUNode no (top=25)				
2	18,572	24,831	27,664	25,831
3	22,317	30,845	34,966	32,613
4	24,211	33,763	38,158	35,857
5	26,162	35,914	40,116	38,143
6	27,304	36,991	40,833	39,258
7	27,967	37,557	40,928	39,764
8	28,225	37,745	40,613	39,739
9	29,300	38,515	41,035	40,345
10	29,976	38,848	41,198	40,592
11	30,173	39,001	41,264	40,697
12	29,875	38,735	40,984	40,415
13	30,437	39,103	41,326	40,772
14	30,582	39,150	41,205	40,759
15	30,509	38,986	40,782	40,520
16	29,857	38,482	39,990	40,005
17	29,951	38,550	39,746	40,153
18	29,519	38,243	39,094	40,085
19	28,674	37,617	38,068	39,785
20	27,211	36,474	36,449	38,917
21	26,148	35,470	34,847	38,102
22	24,196	33,379	32,126	35,972
23	21,218	29,888	27,929	31,934
24	16,789	24,090	22,053	25,653
25	12,124	18,330	9,013	10,199

Void history	5535	11494	11495	13775
Node no (top=25)				
1	-0.04	-0.03	-0.03	-0.03
2	-0.04	-0.03	-0.03	-0.03
3	-0.03	-0.01	-0.01	-0.01
4	0.00	0.04	0.04	0.05
5	0.05	0.11	0.11	0.12
6	0.10	0.19	0.19	0.20
7	0.14	0.26	0.26	0.27
8	0.17	0.32	0.32	0.33
9	0.19	0.37	0.37	0.39
10	0.21	0.43	0.43	0.44
11	0.23	0.47	0.47	0.48
12	0.25	0.51	0.51	0.52
13	0.27	0.55	0.55	0.56
14	0.29	0.58	0.58	0.59
15	0.31	0.61	0.61	0.61
16	0.33	0.63	0.63	0.64
17	0.35	0.66	0.66	0.66
18	0.37	0.68	0.68	0.68
19	0.39	0.70	0.70	0.69
20	0.40	0.71	0.71	0.71
21	0.41	0.73	0.73	0.72
22	0.42	0.74	0.74	0.73
23	0.43	0.75	0.75	0.74
24	0.44	0.75	0.75	0.74
25	0.46	0.76	0.76	0.75
Density history				
Node no (top=25)				
1	0.77	0.76	0.76	0.76
2	0.76	0.76	0.76	0.76
3	0.75	0.74	0.74	0.74
4	0.72	0.70	0.70	0.71
5	0.69	0.66	0.66	0.67
6	0.66	0.61	0.61	0.62
7	0.63	0.57	0.57	0.58
8	0.61	0.52	0.52	0.54
9	0.60	0.48	0.48	0.50
10	0.58	0.45	0.45	0.47
11	0.57	0.42	0.42	0.44
12	0.56	0.39	0.39	0.41
13	0.55	0.36	0.36	0.39
14	0.53	0.34	0.34	0.36
15	0.51	0.32	0.32	0.34
16	0.50	0.31	0.31	0.33
17	0.49	0.29	0.29	0.31
18	0.47	0.28	0.28	0.30
19	0.46	0.26	0.26	0.28
20	0.46	0.25	0.25	0.27
21	0.45	0.24	0.24	0.26
22	0.44	0.24	0.24	0.26
23	0.44	0.23	0.23	0.25
24	0.43	0.22	0.22	0.24
25	0.42	0.22	0.22	0.24

Control rod history
Node no (top=25)

1	0.00	0.03	0.03	0.00
2	0.00	0.02	0.02	0.00
3	0.00	0.02	0.02	0.00
4	0.00	0.01	0.01	0.00
5	0.00	0.01	0.01	0.00
6	0.00	0.01	0.01	0.00
7	0.00	0.01	0.01	0.00
8	0.00	0.01	0.01	0.00
9	0.00	0.01	0.01	0.00
10	0.00	0.01	0.01	0.00
11	0.00	0.01	0.01	0.00
12	0.00	0.01	0.01	0.00
13	0.00	0.01	0.01	0.00
14	0.00	0.01	0.01	0.00
15	0.00	0.00	0.00	0.00
16	0.00	0.00	0.00	0.00
17	0.00	0.00	0.00	0.00
18	0.00	0.00	0.00	0.00
19	0.00	0.00	0.00	0.00
20	0.00	0.00	0.00	0.00
21	0.00	0.00	0.00	0.00
22	0.00	0.00	0.00	0.00
23	0.00	0.00	0.00	0.00
24	0.00	0.00	0.00	0.00
25	0.00	0.00	0.00	0.00

Forsmark 2 Enrichment diagram assembly 5535 (without BA)



Forsmark 2 Fuel rod diagram assembly 11494



PATRON NR 11494
BOX NR 34S024

ANRIKN %	ANTAL	KG	KG	KG
VERKL	STAVAR	UO2	UTOT	U235
2.920	63	205.704	181.088	5.287

	076-	076-	076-	076-	076-	076-	076-	076-
8	20224	24651	24509	24549	24556	24538	24558	20221
	1.98	2.48	2.48	2.48	2.48	2.48	2.48	1.98
	3263	3263	3260	3261	3261	3255	3262	3265
7	24553	15458	00604	21902	21951	00639	15485	24528
	2.48	3.17	3.32	3.32	3.32	3.32	3.17	2.48
	3259	3239	3278	3271	3275	3270	3235	3260
6	076-	076-	076-	076-	076-	076-	076-	076-
	24550	00610	00718	21907	21993	00708	00635	24709
	2.48	3.32	3.32	3.32	3.32	3.32	3.32	2.48
	3261	3274	3268	3274	3275	3267	3272	3256
5	076-	076-	076-	076-	076-	076-	076-	076-
	24546	21971	21920	21852	21840	21955	21979	24589
	2.48	3.32	3.32	3.32	3.32	3.32	3.32	2.48
	3260	3266	3271	3277	3272	3267	3275	3266
4	076-	076-	076-	076-	076-	076-	076-	076-
	24639	21906	21944	21918	21879	21903	21926	24529
	2.48	3.32	3.32	3.32	3.32	3.32	3.32	2.48
	3268	3272	3280	3273	3267	3271	3270	3259
3	076-	076-		076-	076-	076-	076-	076-
	24615	00665		21970	21972	00728	00663	24631
	2.48	3.32		3.32	3.32	3.32	3.32	2.48
	3261	3271		3273	3273	3265	3270	3264
2	076-	076-	076-	076-	076-	076-	076-	076-
	24552	15453	00603	21989	21991	00657	15474	24540
	2.48	3.17	3.32	3.32	3.32	3.32	3.17	2.48
	3265	3230	3277	3274	3277	3267	3238	3253
1	076-	076-	076-	076-	076-	076-	076-	076-
	20211	24554	24590	24614	24609	24617	24580	20180
	1.98	2.48	2.48	2.48	2.48	2.48	2.48	1.98
	3269	3266	3263	3261	3266	3258	3263	3262
	A	B	C	D	E	F	G	H

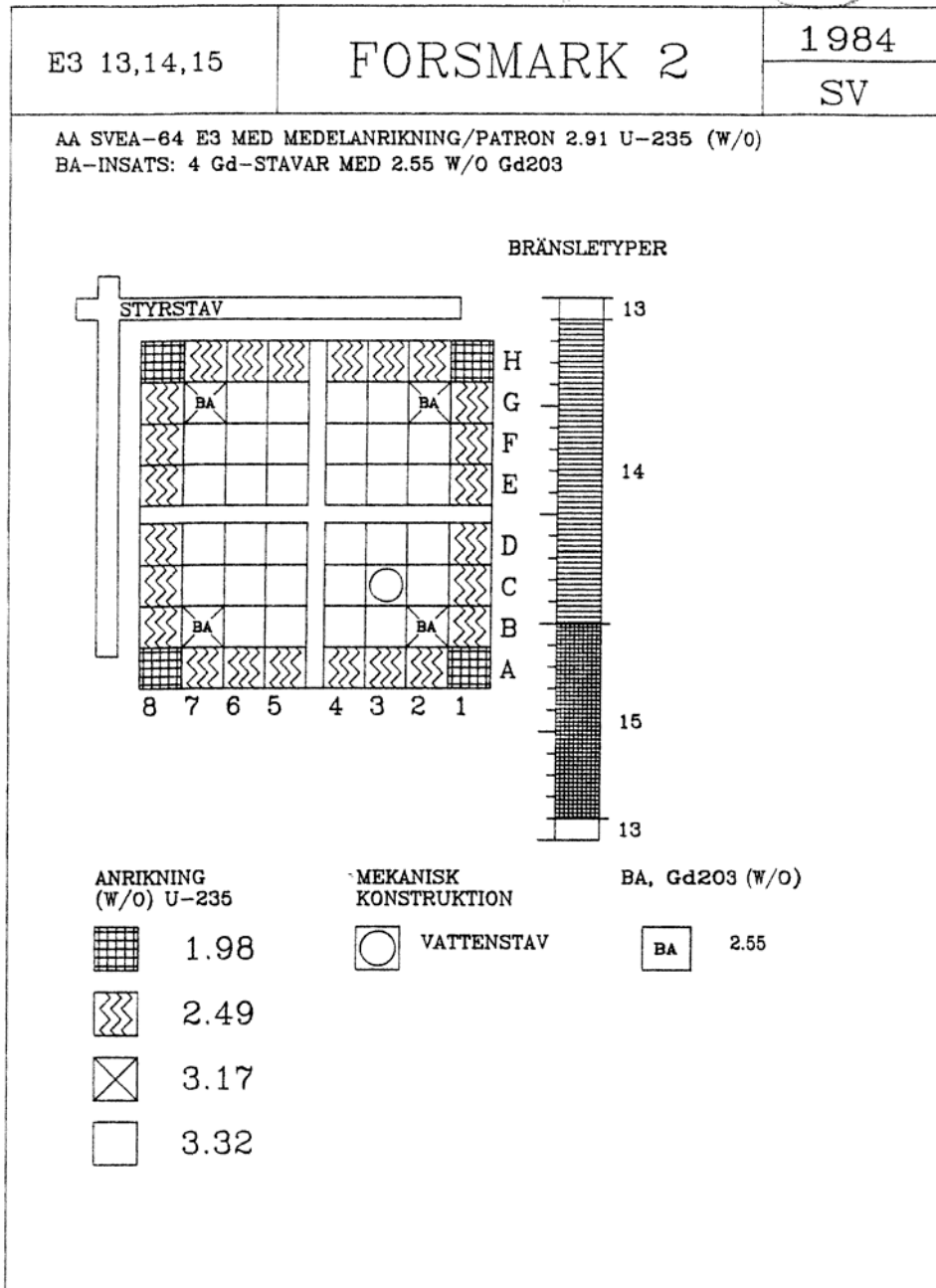
Forsmark 2 Fuel rod diagram assembly 11495



PATRON NR 11495
BOX NR 34S042

ANRIKN %	ANTAL	KG	KG	KG
VERKL	STAVAR	UO2	UTOT	U235
2.920	63	205.684	181.070	5.287

	076-	076-	076-	076-	076-	076-	076-	076-
	20204	24572	24519	24534	24624	24555	24655	20184
8	1.98	2.48	2.48	2.48	2.48	2.48	2.48	1.98
	3264	3258	3256	3264	3264	3265	3267	3266
	076-	076-	076-	076-	076-	076-	076-	076-
	24563	15479	00623	21839	21823	00654	15491	24566
7	2.48	3.17	3.32	3.32	3.32	3.32	3.17	2.48
	3259	3234	3275	3275	3277	3267	3241	3261
	076-	076-	076-	076-	076-	076-	076-	076-
	24571	00636	00729	21847	21820	00717	00664	24542
6	2.48	3.32	3.32	3.32	3.32	3.32	3.32	2.48
	3256	3267	3265	3267	3278	3266	3271	3259
	076-	076-	076-	076-	076-	076-	076-	076-
	24524	21807	21868	21832	21867	21801	21934	24578
5	2.48	3.32	3.32	3.32	3.32	3.32	3.32	2.48
	3257	3277	3271	3273	3272	3278	3276	3254
	076-	076-	076-	076-	076-	076-	076-	076-
	24515	21822	21893	21887	21806	21889	21896	24511
4	2.48	3.32	3.32	3.32	3.32	3.32	3.32	2.48
	3257	3278	3272	3272	3277	3271	3271	3262
	076-	076-		076-	076-	076-	076-	076-
	24569	00605		21953	21884	00701	00651	24573
3	2.48	3.32		3.32	3.32	3.32	3.32	2.48
	3265	3277		3277	3274	3266	3268	3259
	076-	076-	076-	076-	076-	076-	076-	076-
	24712	15473	00655	21805	21875	00621	15452	24536
2	2.48	3.17	3.32	3.32	3.32	3.32	3.17	2.48
	3256	3236	3266	3267	3274	3272	3237	3256
	076-	076-	076-	076-	076-	076-	076-	076-
	20214	24595	24643	24567	24564	24557	24582	20213
1	1.98	2.48	2.48	2.48	2.48	2.48	2.48	1.98
	3262	3260	3265	3260	3260	3261	3262	3264
	A	B	C	D	E	F	G	H



Forsmark 2 Fuel rod diagram assembly 13775



PATRON NR 13775
BOX NR 44S196

ANRIKN %	ANTAL	KG	KG	KG
VERKL	STAVAR	UO2	UTOT	U235
2.854	63	206.067	181.340	5.175

	099-	099-	099-	099-	099-	099-	099-	099-
8	20698	20967	21040	21009	21029	20972	20931	20834
	1.98	2.48	2.48	2.48	2.48	2.48	2.48	1.98
	3266	3268	3266	3267	3273	3275	3271	3270
7	099-	099-	099-	099-	099-	099-	099-	099-
	21044	15422	114	20631	20658	185	15424	20962
	2.48	3.14	3.29	3.29	3.29	3.29	3.14	2.48
	3265	3253	3277	3274	3274	3263	3252	3271
6	099-	099-	099-	099-	099-	099-	099-	099-
	21056	202	1017	20602	20567	1041	173	20927
	2.48	3.29	3.29	3.29	3.29	3.29	3.29	2.48
	3274	3273	3278	3273	3279	3278	3271	3271
5	099-	099-	099-	099-	099-	099-	099-	099-
	20947	20635	20592	20594	15406	20572	20670	20971
	2.48	3.29	3.29	3.29	3.14	3.29	3.29	2.48
	3274	3281	3278	3274	3248	3274	3274	3275
4	099-	099-	099-	099-	099-	099-	099-	099-
	20998	20587	20553	20534	20623	20624	20484	21050
	2.48	3.29	3.29	3.29	3.29	3.29	3.29	2.48
	3279	3278	3274	3271	3274	3271	3273	3278
3	099-	099-		099-	099-	099-	099-	099-
	20938	067		20483	20625	1038	084	20990
	2.48	3.29		3.29	3.29	3.29	3.29	2.48
	3273	3277		3276	3273	3275	3287	3254
2	099-	099-	099-	099-	099-	099-	099-	099-
	21047	15270	001	20532	20550	044	15266	21021
	2.48	3.14	3.29	3.29	3.29	3.29	3.14	2.48
	3276	3251	3264	3271	3270	3279	3247	3266
1	099-	099-	099-	099-	099-	099-	099-	099-
	20729	21032	20923	20991	20997	21094	20941	20711
	1.98	2.48	2.48	2.48	2.48	2.48	2.48	1.98
	3273	3279	3267	3270	3266	3273	3274	3268
	A	B	C	D	E	F	G	H

Forsmark 2 Fuel rod diagram assembly 13775



PATRON NR 13775
BOX NR 44S196

ANRIKN %	ANTAL	KG	KG	KG
VERKL	STAVAR	UO2	UTOT	U235
2.854	63	206.067	181.340	5.175

		099- 20698	099- 20967	099- 21040	099- 21009	099- 21029	099- 20972	099- 20931	099- 20834
8		1.98	2.48	2.48	2.48	2.48	2.48	2.48	1.98
		3266	3268	3266	3267	3273	3275	3271	3270
		099- 21044	099- 15422	099- 114	099- 20631	099- 20658	099- 185	099- 15424	099- 20962
7		2.48	3.14	3.29	3.29	3.29	3.29	3.14	2.48
		3265	3253	3277	3274	3274	3263	3252	3271
		099- 21056	099- 202	099- 1017	099- 20602	099- 20567	099- 1041	099- 173	099- 20927
6		2.48	3.29	3.29	3.29	3.29	3.29	3.29	2.48
		3274	3273	3278	3273	3279	3278	3271	3271
		099- 20947	099- 20635	099- 20592	099- 20594	099- 15406	099- 20572	099- 20670	099- 20971
5		2.48	3.29	3.29	3.29	3.14	3.29	3.29	2.48
		3274	3281	3278	3274	3248	3274	3274	3275
		099- 20998	099- 20587	099- 20553	099- 20534	099- 20623	099- 20624	099- 20484	099- 21050
4		2.48	3.29	3.29	3.29	3.29	3.29	3.29	2.48
		3279	3278	3274	3271	3274	3271	3273	3278
		099- 20938	099- 067		099- 20483	099- 20625	099- 1038	099- 084	099- 20990
3		2.48	3.29		3.29	3.29	3.29	3.29	2.48
		3273	3277		3276	3273	3275	3287	3254
		099- 21047	099- 15270	099- 001	099- 20532	099- 20550	099- 044	099- 15266	099- 21021
2		2.48	3.14	3.29	3.29	3.29	3.29	3.14	2.48
		3276	3251	3264	3271	3270	3279	3247	3266
		099- 20729	099- 21032	099- 20923	099- 20991	099- 20997	099- 21094	099- 20941	099- 20711
1		1.98	2.48	2.48	2.48	2.48	2.48	2.48	1.98
		3273	3279	3267	3270	3266	3273	3274	3268
		A	B	C	D	E	F	G	H

2.3 Forsmark 3

REACTOR FORSMARK 3

Reactor data	Value
Reactor pressure (MPa) at full power	7
Inlet temperature into core (C) at full power	260
Outlet temperature into core (C) at full power	286
Pitch between assemblis in the core (c-c, mm)	
Avg. temperature in the uranium pellets at full power (C)	507
Avg. temperature in the cladding at full power (C)	290
Temperature in the boxwall at full power (C)	286

Cycle data

Cycle no	Start	Stop	Reactor power MW	Burnup avg/cycle MWd/tU	No of assemblies in core
1a	1984-12-01	1985-06-21	3,020	804	700
1b	1985-07-16	1986-08-08	3,020	8,544	700
2	1986-09-02	1987-07-10	3,020	6,536	700
3	1987-08-09	1988-08-12	3,300	7,906	700
4	1988-09-05	1989-06-09	3,300	5,686	700
5	1989-07-13	1990-07-13	3,300	7,643	700
6	1990-07-29	1991-08-17	3,300	8,984	700
7	1991-09-02	1992-05-15	3,300	6,343	700
8	1992-06-16	1993-06-05	3,300	8,002	700
9	1993-06-21	1994-07-09	3,300	9,148	700
10	1994-07-27	1995-07-29	3,300	9,094	700
11	1995-08-16	1996-05-16	3,300	6,940	700
12	1996-06-17	1997-07-05	3,300	9,816	700

Fuel data	Fuel id 13847	13848
Fuel type	Svea 100	Svea100
No of fuel rods	100	100
Rod pitch normal rods (mm)	12.4	12.4
Rod pitch normal – corner rods (mm)		
Rod pitch corner – corner rods (mm)		
Rod diameter normal rod (mm)	12.4	12.4
Clad thickness normal rod (mm)	9.62	9.62
Pellet diameter normal rod (mm)	0.63	0.63
No of normal rods	8.19	8.19
	100	100
Rod diameter corner rod (mm)		
Clad thickness corner rod (mm)		
Pellet diameter corner rod (mm)		
No of corner rods		
Active length (mm)	3,680	3,680
Density UO2 (g/cc)	10.7	10.7
Porsity, dishing etc (%)	1.09	1.09
No of spacer rods		
Material in spacer rods		
Outer diameter spacer rods (mm)		
Cladding thickness spacer rod		
No of water rods		
Material in water rods		
Outer diameter water rods (mm)		
Inner diameter water rods (mm)		
No of BA rods	8	8
% Gd2O3	4.4	4.4
Rel poison	1	1
No of spacers	4x6	4x6
Spacer material	Inconel	Inconel
Mass of spacer (g)	35	35
Box material	Zr4	Zr4
Box outer measure square (mm)	139.6	139.6

Initial data	13847	13848
Initial mass Utot (g)	180,669	180,667
Initial mass U235 (g)	5,002	5,002
Avg.enrichment% U235	2.769	2.769
Data after rebuild 1		
Date of rebuild		
No of fuel rods		
No spacer rods		
No water rods		
No of water holes		
Mass Utot after rebuild (g)		
Mass U235 after rebuild (g)		
Data after rebuild 2		
Date of rebuild		
No of fuel rods		
No spacer rods		
No water rods		
No of water holes		
Mass Utot after rebuild (g)		
Mass U235 after rebuild (g)		
Cycle history burnup/cycle, MWd/tU		
1a		
1b		
2	7,857	7,857
3	9,360	9,360
4	6,562	6,562
5	7,496	7,496
6		
Axial BU distribution, EOL MWd/tUNode no (top=25)		
1	7,519	7,519
2	24,984	24,984
3	30,453	30,453
4	32,711	32,711
5	34,413	34,413
6	35,093	35,093
7	35,289	35,289
8	35,041	35,041
9	35,606	35,606
10	35,824	35,824
11	35,873	35,873
12	35,478	35,478
13	35,803	35,803
14	35,762	35,762
15	35,526	35,526
16	35,014	35,014
17	35,262	35,262
18	35,157	35,157
19	34,738	34,738
20	33,746	33,746
21	32,941	32,941
22	30,977	30,977
23	27,464	27,464
24	22,109	22,109
25	9,084	9,084

Void history	13847	13848
Node no (top=25)		
1	-0.02	-0.02
2	-0.02	-0.02
3	0.00	0.00
4	0.06	0.06
5	0.14	0.14
6	0.22	0.22
7	0.30	0.30
8	0.36	0.36
9	0.42	0.42
10	0.47	0.47
11	0.51	0.51
12	0.55	0.55
13	0.58	0.58
14	0.61	0.61
15	0.64	0.64
16	0.66	0.66
17	0.68	0.68
18	0.70	0.70
19	0.72	0.72
20	0.73	0.73
21	0.74	0.74
22	0.76	0.76
23	0.76	0.76
24	0.77	0.77
25	0.77	0.77
Density history		
Node no (top=25)		
1	757	757
2	753	753
3	730	730
4	686	686
5	633	633
6	580	580
7	530	530
8	486	486
9	448	448
10	414	414
11	384	384
12	359	359
13	337	337
14	317	317
15	300	300
16	285	285
17	271	271
18	259	259
19	248	248
20	238	238
21	229	229
22	221	221
23	215	215
24	210	210
25	206	206

Control rod history
Node no (top=25)

1	0.02	0.02
2	0.02	0.02
3	0.01	0.01
4	0	0
5	0	0
6	0	0
7	0	0
8	0	0
9	0	0
10	0	0
11	0	0
12	0	0
13	0	0
14	0	0
15	0	0
16	0	0
17	0	0
18	0	0
19	0	0
20	0	0
21	0	0
22	0	0
23	0	0
24	0	0
25	0	0

Forsmark 3 Fuel rod diagram assembly 13847



PATRON NR 13847
BOX NR 44S399

ANRIKN % VERKL	ANTAL STAVAR	KG UO2	KG UTOT	KG U235
2.769	100	205.738	180.669	5.002

	111-	111-	111-	111-	111-	111-	111-	111-	111-	111-
	20059	20181	20207	20103	20419	20358	20159	20191	20099	20098
10	1.91	2.41	2.41	2.41	2.41	2.41	2.41	2.41	1.91	1.64
	2060	2060	2063	2065	2060	2058	2065	2063	2060	2056
	111-	111-	111-	111-	111-	111-	111-	111-	111-	111-
	20003	20414	15024	27	20289	20286	16	15023	20065	20039
9	2.41	3.08	3.12	3.08	3.08	3.08	3.08	3.12	2.41	1.91
	2067	2055	2038	2055	2056	2057	2059	2040	2065	2060
	111-	111-	111-	111-	111-	111-	111-	111-	111-	111-
	20137	20444	1034	20445	20353	20236	20227	1026	15012	20263
8	2.41	3.08	3.08	3.08	3.08	3.08	3.08	3.08	3.12	2.41
	2058	2057	2063	2058	2059	2058	2054	2057	2040	2059
	111-	111-	111-	111-	111-	111-	111-	111-	111-	111-
	20048	51	20310	20415	20380	20325	20341	20356	45	20222
7	2.41	3.08	3.08	3.08	3.08	3.08	3.08	3.08	3.08	2.41
	2060	2056	2059	2059	2057	2059	2058	2055	2059	2062
	111-	111-	111-	111-	111-	111-	111-	111-	111-	111-
	20192	15025	20335	20401	20397	20328	20373	20261	20302	20186
6	2.41	3.12	3.08	3.08	3.08	3.08	3.08	3.08	3.08	2.41
	2061	2039	2058	2058	2057	2058	2055	2056	2057	2063
	111-	111-	111-	111-	111-	111-	111-	111-	111-	111-
	20043	20426	20294	20547	15051	20233	20265	20322	20248	20202
5	2.41	3.08	3.08	3.08	3.12	3.08	3.08	3.08	3.08	2.41
	2065	2059	2059	2057	2042	2055	2058	2057	2054	2062
	111-	111-	111-	111-	111-	111-	111-	111-	111-	111-
	20420	29	20421	20436	20342	20403	20410	20367	56	20150
4	2.41	3.08	3.08	3.08	3.08	3.08	3.08	3.08	3.08	2.41
	2058	2058	2057	2057	2057	2058	2058	2058	2059	2062
	111-	111-	111-	111-	111-	111-	111-	111-	111-	111-
	20031	20434	1014	20361	20383	20385	20370	1032	15027	20363
3	2.41	3.08	3.08	3.08	3.08	3.08	3.08	3.08	3.12	2.41
	2063	2059	2059	2061	2055	2055	2057	2062	2043	2059
	111-	111-	111-	111-	111-	111-	111-	111-	111-	111-
	20443	15007	20247	36	20362	15041	57	20351	20448	20114
2	2.41	3.12	3.08	3.08	3.08	3.12	3.08	3.08	3.08	2.41
	2059	2041	2059	2060	2057	2043	2058	2055	2058	2062
	111-	111-	111-	111-	111-	111-	111-	111-	111-	111-
	20213	20252	20131	20433	20055	20154	20140	20311	20163	20195
1	1.91	2.41	2.41	2.41	2.41	2.41	2.41	2.41	2.41	1.91
	2055	2060	2062	2059	2063	2058	2061	2058	2059	2057
	A	B	C	D	E	F	G	H	I	J

Forsmark 3 Fuel rod diagram assembly 13848

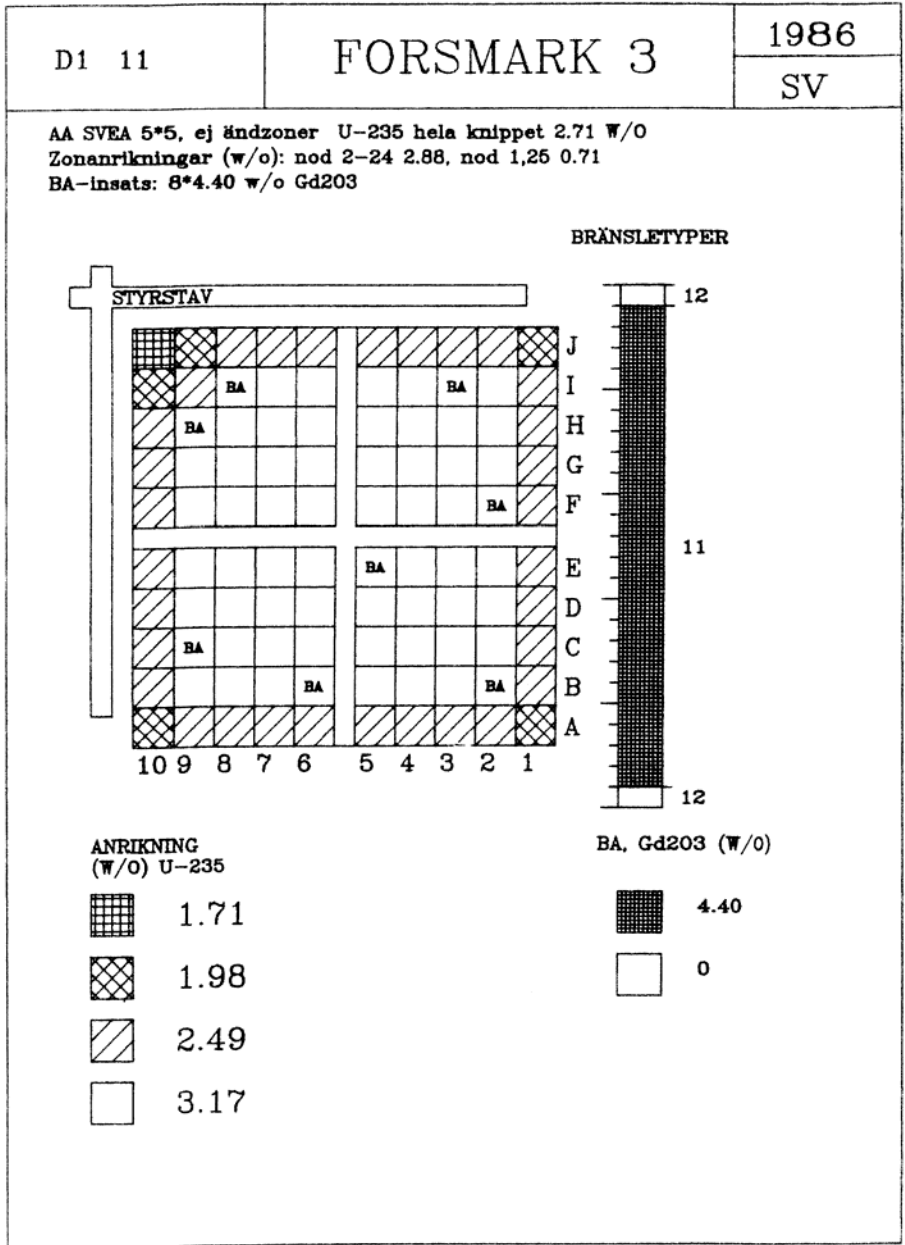


PATRON NR 13848
BOX NR 44S408

ANRIKN %	ANTAL	KG	KG	KG
VERKL	STAVAR	UO2	UTOT	U235
2.769	100	205.734	180.667	5.002

	111-	111-	111-	111-	111-	111-	111-	111-	111-	
	20071	20117	20122	20442	20112	20146	20387	20021	20060	20002
10	1.91	2.41	2.41	2.41	2.41	2.41	2.41	2.41	1.91	1.64
	2057	2061	2065	2060	2064	2064	2060	2062	2057	2055
	111-	111-	111-	111-	111-	111-	111-	111-	111-	111-
	20125	20339	15031	48	20424	20231	54	15015	20051	20092
9	2.41	3.08	3.12	3.08	3.08	3.08	3.08	3.12	2.41	1.91
	2060	2057	2042	2059	2058	2058	2058	2043	2061	2062
	111-	111-	111-	111-	111-	111-	111-	111-	111-	111-
	20221	20296	1021	20388	20273	20226	20281	1020	15005	20249
8	2.41	3.08	3.08	3.08	3.08	3.08	3.08	3.08	3.12	2.41
	2056	2065	2065	2058	2058	2056	2058	2061	2041	2061
	111-	111-	111-	111-	111-	111-	111-	111-	111-	111-
	20023	49	20441	20438	20450	20259	20285	20425	9	20132
7	2.41	3.08	3.08	3.08	3.08	3.08	3.08	3.08	3.08	2.41
	2059	2058	2058	2057	2056	2056	2055	2057	2059	2062
	111-	111-	111-	111-	111-	111-	111-	111-	111-	111-
	20429	15017	20327	20354	20276	20301	20406	20306	20416	20111
6	2.41	3.12	3.08	3.08	3.08	3.08	3.08	3.08	3.08	2.41
	2059	2032	2058	2058	2055	2058	2058	2056	2055	2065
	111-	111-	111-	111-	111-	111-	111-	111-	111-	111-
	20115	20355	20369	20277	15008	20288	20239	20366	20232	20217
5	2.41	3.08	3.08	3.08	3.12	3.08	3.08	3.08	3.08	2.41
	2061	2059	2056	2056	2044	2059	2057	2055	2059	2064
	111-	111-	111-	111-	111-	111-	111-	111-	111-	111-
	20393	50	20293	20382	20314	20260	20374	20430	10	20158
4	2.41	3.08	3.08	3.08	3.08	3.08	3.08	3.08	3.08	2.41
	2059	2060	2059	2058	2058	2056	2056	2054	2058	2063
	111-	111-	111-	111-	111-	111-	111-	111-	111-	111-
	20020	20235	1008	20412	20228	20272	20299	1003	15053	20061
3	2.41	3.08	3.08	3.08	3.08	3.08	3.08	3.08	3.12	2.41
	2062	2058	2063	2058	2058	2058	2056	2063	2042	2065
	111-	111-	111-	111-	111-	111-	111-	111-	111-	111-
	20026	15037	20313	37	20439	15004	43	20329	20291	20012
2	2.41	3.12	3.08	3.08	3.08	3.12	3.08	3.08	3.08	2.41
	2059	2040	2058	2055	2054	2037	2060	2058	2054	2060
	111-	111-	111-	111-	111-	111-	111-	111-	111-	111-
	20200	20022	20219	20127	20331	20147	20001	20025	20129	20053
1	1.91	2.41	2.41	2.41	2.41	2.41	2.41	2.41	2.41	1.91
	2054	2063	2064	2062	2058	2060	2057	2064	2061	2058
	A	B	C	D	E	F	G	H	I	J

5.13



2.4 Oskarshamn 2

REACTOR OSKARSHAMN 2

Reactor data	Value
Reactor pressure (MPa) at full power	7
Inlet temperature into core (C) at full power	274
Outlet temperature into core (C) at full power	286
Avg. temperature in the uranium pellets at full power (C)	500
Avg. temperature in the cladding at full power (C)	290
Temperature in the boxwall at full power (C)	286

Cycle data

Cycle no	Start	Stop	Reactor power MW	Burnup avg/cycle MWd/tU	No of assemblies in core
1	1974-10-02	1976-04-16	3,978	1,800	444
2	1976-07-02	1977-05-13	5,164	1,800	444
3	1977-08-07	1978-06-24	6,210	1,800	444
4	1978-09-03	1979-07-20	6,099	1,800	444
5	1979-08-30	1980-07-12	6,245	1,800	444
6	1980-08-20	1981-07-15	6,474	1,800	444
7	1981-08-30	1982-07-23	6,677	1,800	444
8	1982-08-23	1983-08-19	7,627	1,800	444
9	1983-09-19	1984-07-01	4,784	1,800	444
10	1984-08-24	1985-06-07	7,912	1,800	444
11	1985-07-13	1986-08-16	7,752	1,800	444
12	1986-09-27	1987-07-31	6,429	1,800	444
13	1987-09-04	1988-08-20	6,942	1,800	444
14	1988-09-10	1989-08-05	5,788	1,800	444
15	1989-09-01	1990-08-10	6,578	1,800	444
16	1990-10-11	1991-08-02	6,526	1,800	444

Fuel data

Fuel data	Fule id 1377	1389	1546	1696	1704	2995	3054	3058	3064	6350	12684
Fuel type	8x8	8x8	8x8	8x8	8x8	8x8	8x8	8x8	8x8	8x8	SVEA 64
No of fuel rods	63	63	63	63	63	63	63	63	63	63	63
Rod pitch normal rods (mm)	16.3	16.3	16.3	16.3	16.3	16.3	16.3	16.3	16.3	16.3	15.8
Rod pitch normal – corner rods (mm)	16.05	16.05	16.05	16.05	16.05	16.05	16.05	16.05	16.05	16.05	
Rod pitch corner – corner rods (mm)	15.8	15.8	15.8	15.8	15.8	15.8	15.8	15.8	15.8	15.8	
Rod diameter normal rod (mm)	12.25	12.25	12.25	12.25	12.25	12.25	12.25	12.25	12.25	12.25	12.25
Clad thickness normal rod (mm)	0.8	0.8	0.8	0.8	0.8	0.8	0.8	0.8	0.8	0.8	0.8
Pellet diameter normal rod (mm)	10.44	10.44	10.44	10.44	10.44	10.44	10.44	10.44	10.44	10.44	10.44
No of normal rods	51	51	51	51	51	51	51	51	51	51	63
Rod diameter corner rod (mm)	11.75	11.75	11.75	11.75	11.75	11.75	11.75	11.75	11.75	11.75	11.75
Clad thickness corner rod (mm)	0.8	0.8	0.8	0.8	0.8	0.8	0.8	0.8	0.8	0.8	0.8
Pellet diameter corner rod (mm)	9.94	9.94	9.94	9.94	9.94	9.94	9.94	9.94	9.94	9.94	9.94
No of corner rods	12	12	12	12	12	12	12	12	12	12	12
Active length (mm)	3,712	3,712	3,712	3,712	3,712	3,712	3,712	3,712	3,712	3,712	3,712
Density UO ₂ (g/cc)	10.71	10.715	10.733	10.75	10.735	10.465	10.455	10.462	10.46	10.443	10.447
Porsity,dishing etc (%)	1	1	1	1	1	1	1	1	1	1	1
No of spacer rods	1	1	1	1	1	1	1	1	1	(1 tot)	3 (4 tot)
Material in spacer rods	Zr	Zr	Zr	Zr	Zr	Zr	Zr	Zr	Zr	Zr	Zr
Outer diameter spacer rods (mm)	12.25	12.25	12.25	12.25	12.25	12.25	12.25	12.25	12.25		12.25
Cladding thickness spacer rod											0.8
No of water rods										1 sprh	1 sprh
Material in water rods										Zr	Zr
Outer diameter water rods (mm)										12.25	12.25
Inner diameter water rods (mm)										10.65	10.65
No of BA rods	0	0	0	0	0	4	5	5	5	6	6
% Gd ₂ O ₃	0	0	0	0	0	2	2.55	2.55	2.55	3.2	3.15
Rel poison	0	0	0	0	0	1	1	1	1	1	1
No of spacers	6	6	6	6	6	6	6	6	6	6	4x6
Spacer material	Inconel	Inconel	Inconel	Inconel	Inconel	Inconel	Inconel	Inconel	Inconel	Inconel	Inconel
Mass of spacer (g)	135	135	135	135	135	135	135	135	135	150	31
Box material	Zry-4	Zry-4	Zry-4	Zry-4	Zry-4	Zry-4	Zry-4	Zry-4	Zry-4	Zry-4	Zry-4
Box outer measure square (mm)	138.6	138.6	138.6	138.6	138.6	138.6	138.6	138.6	138.6	138.6	139.6
Box wall thickness (mm)	2.3	2.3	2.3	2.3	2.3	2.3	2.3	2.3	2.3	2.3	1,1/0,8

Initial data	1377	1389	1546	1696	1704	2995	3054	3058	3064	6350	12684
Initial mass Utot (g)	183,575	183,650	183,968	184,253	184,022	179,382	179,193	179,340	179,257	179,003	182,320
Initial mass U235 (g)	4,040	4,042	4,049	4,055	4,050	4,842	4,952	4,956	4,955	5,146	5,291
Avg.enrichment% U235	2.201	2.201	2.201	2.201	2.201	2.699	2.764	2.763	2.764	2.875	2.902
Data after rebuild 1											
Date of rebuild									1980-12-05	1980-12-15	1980-12-17
No of fuel rods										56	56
No spacer rods											56
No water rods											
No of water holes											
Mass Utot after rebuild (g)									160,262	160,372	160,318
Mass U235 after rebuild (g)									4,633	4,637	4,635
Data after rebuild 2											
Date of rebuild											
No of fuel rods											
No spacer rods											
No water rods											
No of water holes											
Mass Utot after rebuild (g)											
Mass U235 after rebuild (g)											
Cycle history burnup/cycle, MWd/tU											
1	8,982	4,315	6,961	4,492	4,587						
2	5,564	2,984		3,412	3,445	6,143					
3		4,529		5,354	5,353	7,217	5,703	6,120	8,133		
4		2,710		3,061	3,254	6,346	5,483	5,883	7,733		
5		2,394				3,872	7,921	7,837	6,038		
6		2,548	6,879			6,401				7,113	
7			7,192	2,183	2,799		6,487	7,935	8,487	7,796	
8			3,438	2,368			9,299	4,212		9,110	
9											
10										3,656	
11											11,134
12											8,898
13											8,152
14											7,311
15											7,589
16											3,564
Axial BU distribution, EOL MWd/tUNode no (top=25)											
1	7,600	11,000	13,000	12,100	11,100	19,500	22,300	20,000	18,000	16,300	23,300
2	12,100	16,700	19,800	18,400	17,000	27,900	31,400	28,500	25,900	24,700	37,600
3	14,800	20,000	23,600	21,700	20,300	32,100	36,200	33,000	30,500	29,000	44,300
4	16,300	21,700	25,600	23,400	22,100	34,100	38,300	35,200	32,800	30,800	47,000
5	17,400	22,700	26,800	24,200	23,000	34,900	39,200	36,300	33,800	31,600	49,400
6	17,600	23,200	27,300	24,600	23,500	35,100	39,600	36,900	34,300	31,800	50,700
7	17,400	23,400	27,400	24,800	23,600	35,000	39,700	37,100	34,400	31,800	51,300
8	17,000	23,500	27,600	24,800	23,500	34,900	39,600	37,200	34,500	31,600	51,200
9	16,800	23,400	27,600	24,700	23,300	34,600	39,400	37,000	34,400	31,600	52,300
10	16,600	23,300	27,500	24,600	23,100	34,200	39,300	36,800	34,400	31,400	52,800
11	16,400	23,000	27,600	24,400	22,800	33,900	39,200	36,600	34,300	31,500	53,000
12	16,300	22,700	28,000	24,100	22,400	33,400	39,000	36,300	34,200	31,200	52,400
13	16,200	22,400	28,200	23,800	22,100	33,000	38,800	36,000	33,900	31,000	53,000
14	16,100	22,100	28,100	23,400	21,700	32,600	38,600	35,700	33,700	30,700	53,100
15	16,000	21,700	28,000	23,100	21,300	32,200	38,400	35,400	33,600	30,300	52,700
16	15,800	21,300	27,900	22,700	21,000	31,900	38,000	34,900	33,300	29,800	51,800
17	15,600	20,800	27,800	22,300	20,600	31,400	37,500	34,400	32,900	29,500	51,900
18	15,300	20,300	27,400	21,800	20,100	30,900	36,800	33,600	32,500	29,000	51,400
19	15,000	19,600	26,600	21,100	19,400	30,200	36,000	32,600	31,800	28,400	50,400
20	14,500	18,700	25,500	20,300	18,700	29,400	35,000	31,500	30,800	27,400	48,500
21	13,800	17,600	24,000	19,100	17,500	28,100	33,400	29,900	29,600	26,200	47,100
22	12,800	16,100	22,000	17,500	16,000	26,100	31,200	27,800	27,600	24,400	44,300
23	11,300	13,900	19,100	15,200	13,800	23,100	27,800	24,400	24,800	21,700	39,600
24	8,900	10,900	15,100	12,000	10,800	18,500	22,400	19,500	20,100	17,600	32,300
25	5,900	6,900	10,200	7,700	7,000	12,600	15,300	13,000	13,700	12,500	24,700

Void history												
Node no (top=25)												
1	0.00	0.00	0.00	0.00	0.00	0.00	0.00	0.00	0.00	0.00	-0.01	-0.03
2	0.01	0.00	0.00	0.00	0.00	0.00	0.01	0.00	0.00	0.02	0.00	-0.02
3	0.04	0.00	0.03	0.01	0.01	0.03	0.02	0.02	0.05	0.04	0.01	
4	0.09	0.02	0.07	0.04	0.03	0.07	0.07	0.06	0.10	0.10	0.08	
5	0.15	0.05	0.11	0.08	0.07	0.13	0.12	0.12	0.16	0.17	0.17	
6	0.22	0.08	0.17	0.12	0.11	0.19	0.19	0.18	0.23	0.24	0.25	
7	0.30	0.11	0.23	0.16	0.16	0.25	0.25	0.24	0.29	0.30	0.31	
8	0.37	0.16	0.28	0.21	0.22	0.31	0.31	0.30	0.35	0.36	0.37	
9	0.44	0.20	0.33	0.25	0.27	0.36	0.37	0.35	0.40	0.40	0.42	
10	0.48	0.24	0.38	0.29	0.32	0.41	0.41	0.40	0.45	0.44	0.47	
11	0.53	0.28	0.43	0.32	0.36	0.45	0.46	0.44	0.49	0.48	0.51	
12	0.56	0.32	0.46	0.35	0.41	0.49	0.50	0.47	0.53	0.52	0.55	
13	0.59	0.35	0.50	0.38	0.44	0.52	0.53	0.50	0.56	0.54	0.58	
14	0.62	0.39	0.53	0.41	0.47	0.55	0.56	0.53	0.59	0.57	0.61	
15	0.64	0.42	0.56	0.44	0.50	0.58	0.58	0.56	0.61	0.59	0.64	
16	0.66	0.45	0.58	0.46	0.53	0.60	0.61	0.58	0.64	0.62	0.66	
17	0.68	0.47	0.60	0.49	0.55	0.62	0.63	0.60	0.66	0.64	0.68	
18	0.70	0.50	0.63	0.51	0.57	0.64	0.65	0.62	0.67	0.65	0.70	
19	0.71	0.52	0.64	0.53	0.59	0.65	0.66	0.64	0.69	0.67	0.72	
20	0.72	0.54	0.66	0.54	0.60	0.67	0.68	0.65	0.70	0.69	0.73	
21	0.73	0.55	0.68	0.56	0.62	0.68	0.69	0.67	0.72	0.70	0.74	
22	0.74	0.56	0.69	0.57	0.63	0.69	0.70	0.68	0.73	0.71	0.75	
23	0.74	0.58	0.70	0.58	0.64	0.70	0.71	0.69	0.73	0.72	0.76	
24	0.75	0.58	0.70	0.59	0.65	0.71	0.72	0.69	0.74	0.72	0.77	
25	0.75	0.59	0.71	0.60	0.65	0.72	0.73	0.70	0.74	0.73	0.77	

Control rod history												
Node no (top=25)												
1	0.12	0.00	0.18	0.00	0.00	0.00	0.00	0.00	0.19	0.00	0.00	
2	0.11	0.00	0.17	0.00	0.00	0.00	0.00	0.00	0.18	0.00	0.00	
3	0.12	0.00	0.16	0.00	0.00	0.00	0.00	0.00	0.16	0.00	0.00	
4	0.13	0.00	0.16	0.00	0.00	0.00	0.00	0.00	0.13	0.00	0.00	
5	0.08	0.00	0.16	0.00	0.00	0.00	0.00	0.00	0.12	0.00	0.00	
6	0.08	0.00	0.15	0.00	0.00	0.00	0.00	0.00	0.12	0.00	0.00	
7	0.07	0.00	0.15	0.00	0.00	0.00	0.00	0.00	0.12	0.00	0.00	
8	0.07	0.00	0.15	0.00	0.00	0.00	0.00	0.00	0.12	0.00	0.00	
9	0.07	0.00	0.14	0.00	0.00	0.00	0.00	0.00	0.12	0.00	0.00	
10	0.07	0.00	0.13	0.00	0.00	0.00	0.00	0.00	0.10	0.00	0.00	
11	0.07	0.00	0.12	0.00	0.00	0.00	0.00	0.00	0.10	0.00	0.00	
12	0.07	0.00	0.11	0.00	0.00	0.00	0.00	0.00	0.10	0.00	0.00	
13	0.06	0.00	0.07	0.00	0.00	0.00	0.00	0.00	0.10	0.00	0.00	
14	0.06	0.00	0.07	0.00	0.00	0.00	0.00	0.00	0.10	0.00	0.00	
15	0.06	0.00	0.06	0.00	0.00	0.00	0.00	0.00	0.10	0.00	0.00	
16	0.06	0.00	0.06	0.00	0.00	0.00	0.00	0.00	0.09	0.00	0.00	
17	0.06	0.00	0.02	0.00	0.00	0.00	0.00	0.00	0.09	0.00	0.00	
18	0.06	0.00	0.01	0.00	0.00	0.00	0.00	0.00	0.08	0.00	0.00	
19	0.06	0.00	0.00	0.00	0.00	0.00	0.00	0.00	0.08	0.00	0.00	
20	0.07	0.00	0.00	0.00	0.00	0.00	0.00	0.00	0.08	0.00	0.00	
21	0.07	0.00	0.00	0.00	0.00	0.00	0.00	0.00	0.06	0.00	0.00	
22	0.04	0.00	0.00	0.00	0.00	0.00	0.00	0.00	0.06	0.00	0.00	
23	0.00	0.00	0.00	0.00	0.00	0.00	0.00	0.00	0.04	0.00	0.00	
24	0.00	0.00	0.00	0.00	0.00	0.00	0.00	0.00	0.01	0.00	0.00	
25	0.00	0.00	0.00	0.00	0.00	0.00	0.00	0.00	0.00	0.00	0.00	

ASBA-ATCRA

9 JAN. 1972

KCA 175

Initialpatron
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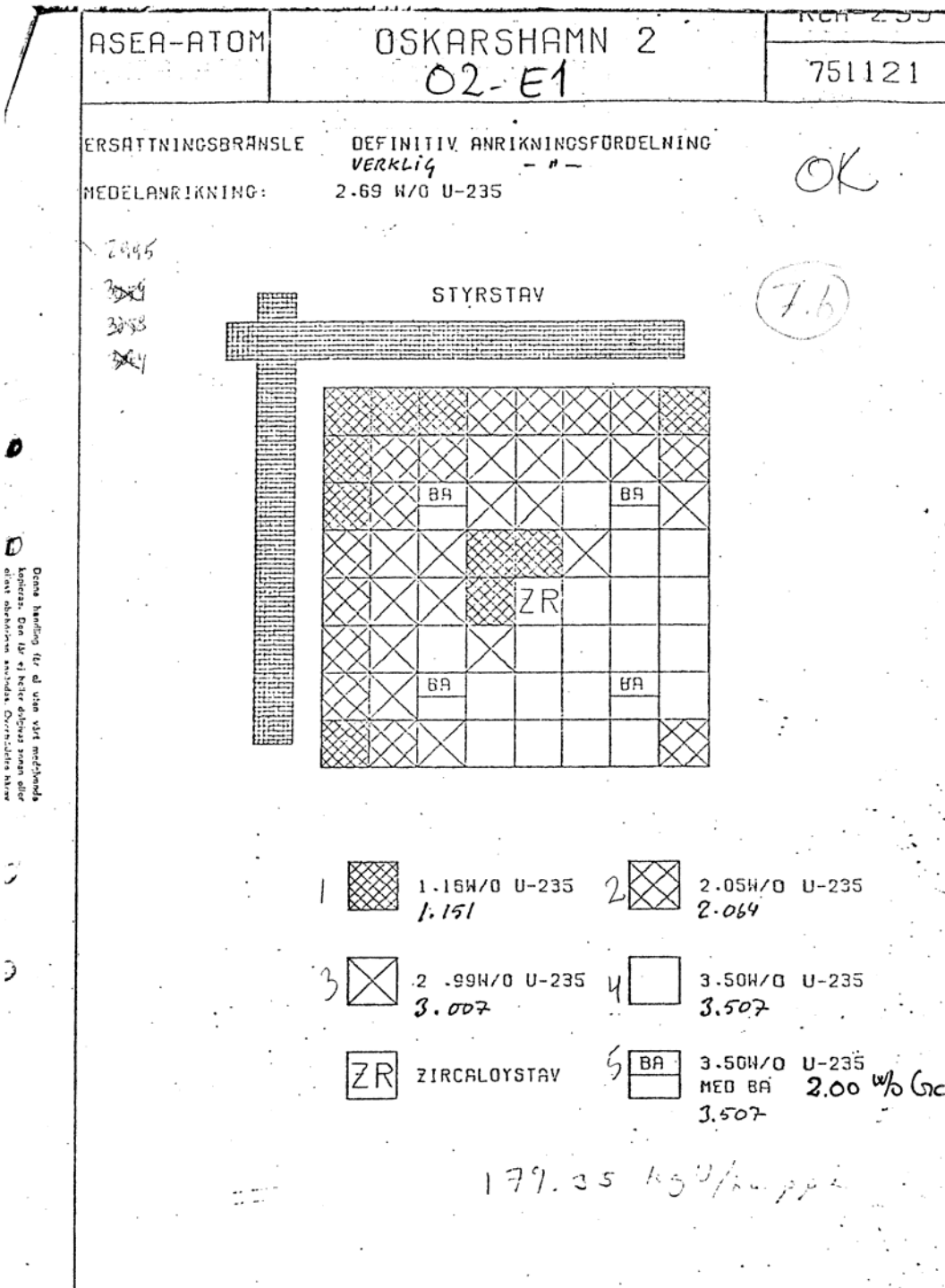
Bred spalt

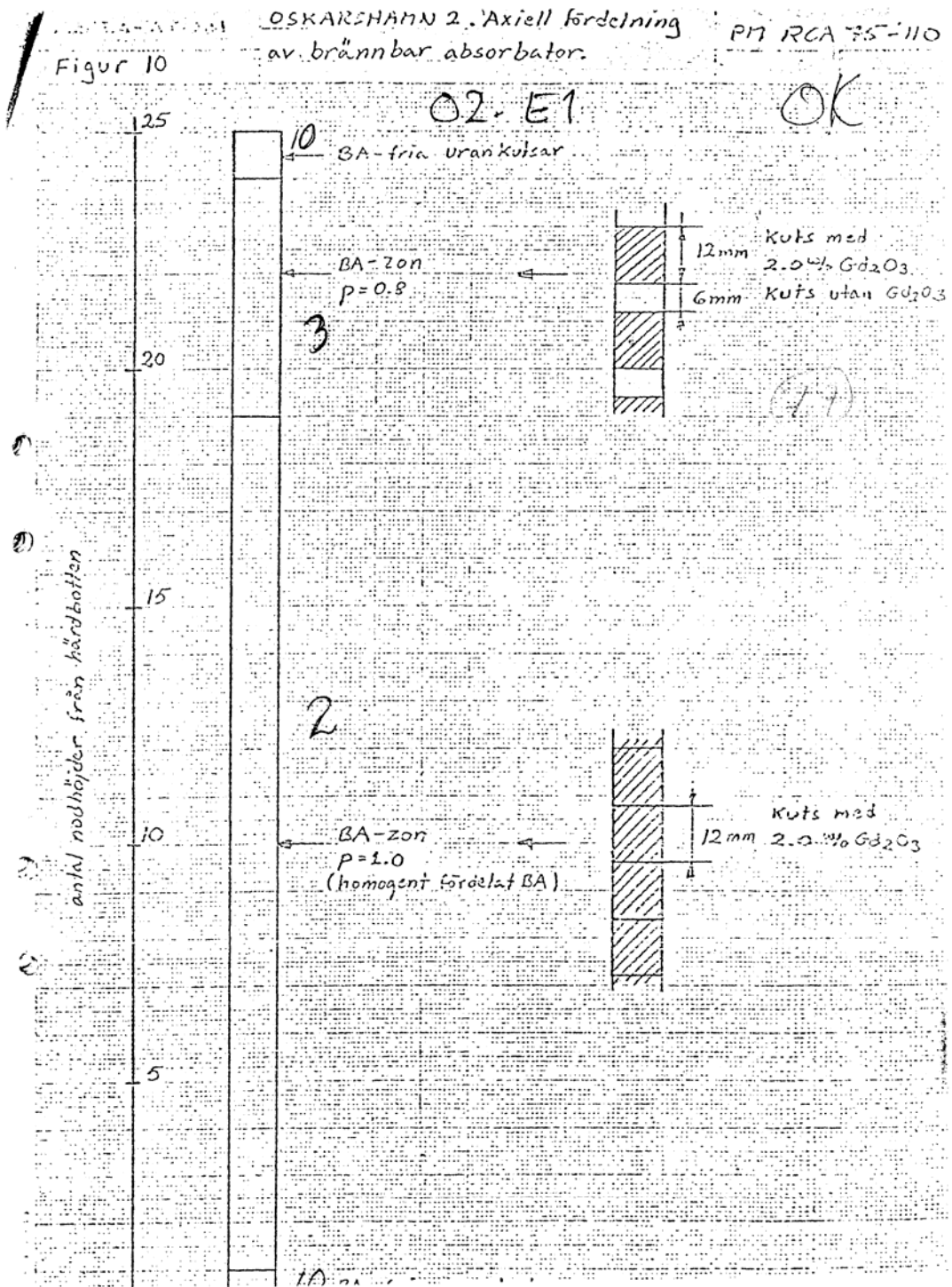
Bred spalt	3	3	3	3	2	2	2	2
	3	2	2	2	2	1	1	1
	3	2	2	1	1	1	1	1
	3	2	1	1	1	1	1	1
	2	2	1	1	1	1	1	1
	2	1	1	1	1	1	1	1
	2	1	1	1	1	1	1	1
	2	1	1	1	1	1	1	2

1-2,5 % U²³⁵
 2-1,65 % U²³⁵
 3-1,4 % U²³⁵

75

Oskarshamn 2 2 Enrichment diagram for assembly 2995





Oskarshamn 2 Fuel rod diagram for assembly 3054

M O N I T E R I N G S P R O T O K O L L

OSKARSHAMN II ERS?

PATRON NR 3054

DATUM: 97-04-77

REF. H

* 2.72	2.02	2.02	2.02	2.02	2.02	1.18	1.18
* 2.979	2.972	3.303	3.309	3.294	3.309	2.972	2.973
8* 5845	5838	20500	20463	20560	20643	5447	5418
* 2.80	2.80	2.80	2.80	3.50BA	2.80	2.02	1.18
* 3.000	3.298	3.295	3.311	3.251	3.295	3.300	2.980
7* 6307	22215	22091	274	15330	22035	20497	5450
* 2.80	3.50BA	3.50	2.80	2.80	3.50	2.80	2.02
* 3.293	3.255	3.297	3.292	3.304	3.293	3.291	3.300
6* 22084	15337	23823	22124	22093	23684	22075	20502
* 3.50	3.50	2.80	1.18	1.18	2.80	3.50BA	2.02
* 3.290	3.298	3.294	3.305	3.303	3.294	3.253	3.304
5* 23639	144	22088	20167	20209	22074	15228	20575
* 3.50	3.50	3.50		1.18	2.80	2.80	2.02
* 3.295	3.291	3.295		3.295	3.297	3.311	3.304
4* 23702	23742	23743	20101	20169	22095	277	20656
* 3.50	3.50	3.50	3.50	2.80	3.50	2.80	2.02
* 3.291	3.298	3.300	3.294	3.291	3.298	3.294	3.305
3* 23649	23691	23756	23745	22051	23736	22061	20663
* 2.80	3.50BA	3.50	3.50	3.50	3.50BA	2.80	2.02
* 2.973	3.249	3.297	3.290	3.302	3.255	3.296	2.967
2* 6283	15346	23729	23787	2	15344	22073	5804
* 2.80	2.80	3.50	3.50	3.50	2.80	2.80	2.02
* 2.994	3.001	3.297	3.296	3.296	3.292	2.999	2.965
1* 6297	6296	23781	23779	23815	22081	6268	5805

A		B		C		D		E		F		G		H	
ANRIKN. %	ANTAL	ANTAL	ANTAL	ANTAL	ANTAL	ANTAL	ANTAL	ANTAL	ANTAL	ANTAL	ANTAL	ANTAL	ANTAL	ANTAL	ANTAL
NOM	VERKL	STAVAR	UO2	U	U-235										
1.18	1.204	6	18.839	16.597	0.200										
2.02	2.031	13	41.616	36.664	0.745										
2.80	2.809	21	67.720	59.661	1.676										
3.50	3.527	18	59.318	52.259	1.843										
3.50BA	3.492	5	16.263	14.012	0.489										
												TOPPL. NR. : 21872			
												BOTTPL. NR. : 21033			
S:A	2.765	63	203.756	179.193	4.953										
SPRIDARE NR.:		9147	7955	7992	7958	7957	8286								
BOX NR.:	3783														
URSPR. LAND FÖR URAN: USA INBLANDAD MÄNGD GD2 O3=4.0%															

Oskarshamn 2 Fuel rod diagram for assembly 3054

M O N T E R I N G S P R O T O K O L L

OSKARSHAMN II ERS2
PATRON NR 3054 DATUM: 97-04-77 REF. H:
Rev 80-12-05

* 2.80	2.80	2.80	2.80	2.80	2.80	2.80	1.18
* 3.000	2.972	3.303	3.303	3.294	3.292	2.973	3.303
* 6307	5838	20500	20463	20560	22124	6283	20209
* 2.80	2.80	2.80	2.80	3.50BA	2.80	2.80	2.80
* 3.294	3.298	3.295	3.311	3.251	3.295	3.300	3.001
* 22088	22215	22091	274	15330	22035	20487	6296
* 2.80	3.50BA	3.50		2.80	3.50	2.80	2.80
* 3.293	3.255	3.297		3.304	3.293	3.291	3.297
* 22084	15337	23823		22093	23684	22075	22095
* 3.50	3.50		1.18		2.80	3.50BA	2.80
* 3.290	3.298		3.305		3.294	3.253	3.304
* 23639	144		20167		22074	15228	20575
* 3.50	3.50	3.50		1.18		2.80	2.80
* 3.295	3.291	3.295		3.295		3.311	3.304
* 23702	23742	23743	20101	20169		277	20656
* 3.50		3.50	3.50		3.50	2.80	2.80
* 3.291		3.300	3.294		3.298	3.294	3.305
* 23649		23756	23745		23736	22061	20663
* 3.50	3.50BA		3.50	3.50	3.50BA	2.80	2.80
* 3.298	3.249		3.290	3.302	3.255	3.296	2.967
* 23691	15346		23787	2	15344	22073	5804
* 2.80	3.50	3.50	3.50	3.50	2.80	2.80	2.80
* 2.994	3.297	3.297	3.296	3.296	3.292	3.291	2.999
* 6297	23729	23781	23779	23815	22081	22051	6268

A	B	C	D	E	F	G	H
ANRIK.N.%	ANTAL	KG	KG	KG			
NOM	VEPKL	STAVAR	U02	U	U-235		
1.18	1.204	3	9.903	8.725	0.105		
2.02	2.031	9	29.063	25.605	0.520		
2.80	2.809	21	67.720	59.651	1.676		
3.50	3.527	18	59.313	52.259	1.843		
3.50BA	3.492	5	16.263	14.012	1.489		
S:A	2.765	56	182.267	160.262	4.653	TOPPL.NR.: 21872	90TTP.L.NP.: 21133
SPRIDARE NR.:		9147	7955	7992	7953	7957	8286
BOX NR.:		3783					
URSPR.LAND FÖR URAN:		USA	INSLANDAD	MÄNGD	GD2	03=4.0%	

Oskarshamn 2 Fuel rod diagram for assembly 3058

OSKARSHAMN II ERG2
DATUM: 12-04-77

PATRON NR 3058 RFF.

```

*****
* 2.02 2.02 2.02 2.02 2.02 2.02 1.18 1.18
* 2.972 2.968 3.304 3.303 3.305 3.309 2.927 2.927
8* 5809 5815 20493 20557 20535 20666 5420 5372
*
*
* 2.80 2.80 2.80 2.80 3.503A 2.80 2.02 1.18
* 2.997 3.306 3.300 3.310 3.256 3.302 3.304 2.999
7* 6301 21816 21820 327 15328 22012 20642 5353
*
*
* 2.80 3.503A 3.50 2.80 2.80 3.50 2.80 2.02
* 3.302 3.254 3.292 3.300 3.310 3.291 3.301 3.307
6* 22000 15261 23695 22014 21801 23818 21824 20644
*
*
* 3.50 3.50 2.80 1.18 1.18 2.80 3.503A 2.02
* 3.292 3.305 3.303 3.306 3.303 3.309 3.252 3.307
5* 23750 47 21996 20200 20223 21907 15376 20641
*
*
* 3.50 3.50 3.50 1.18 2.80 2.80 2.02
* 3.289 3.290 3.298 3.303 3.297 3.303 3.303
4* 23692 23817 23784 21223 20172 21967 417 20674
*
*
* 3.50 3.50 3.50 3.50 2.80 3.50 2.80 2.02
* 3.292 3.299 3.285 3.292 3.303 3.295 3.299 3.307
3* 23791 23772 23819 23321 21864 23738 21973 20673
*
*
* 2.80 3.503A 3.50 3.50 3.50 3.503A 2.80 2.02
* 3.003 3.251 3.290 3.298 3.283 3.252 3.305 2.976
2* 6194 15373 23805 23786 179 15372 21854 5852
*
*
* 2.80 2.80 3.50 3.50 3.50 2.80 2.80 2.02
* 3.005 3.004 3.298 3.299 3.293 3.302 3.001 2.984
1* 6244 6311 23806 23762 23761 21813 6248 5773
*****

```

	A	B	C	D	E	F	G	H
ANRIKN.%								
NOM	1.18	2.02	2.80	3.50	3.50	3.50	3.503A	2.02
VEPKL	1.204	13	21	18	5			
STAVAR	6							
KG								
UO2	18.871	41.654	67.862	59.271	16.265			
U	16.625	36.697	59.736	52.218	14.014			
U-235	0.200	0.745	1.680	1.842	0.489			
S:A	2.765	63	203.923	179.340	4.956			
SPRIDARE NR.:		7963	7966	7996	8106	7988	8361	
BOX NR.:	3852							
URSPR.LAND FÖR URAN:	USA INBLANDAD MÄNGD GD2 O3=4.0%							
TOPPL.NR.:	22244							
BOTTPL.NR.:	2792							

Oskarshamn 2 Fuel rod diagram for assembly 3058

M O N T E R I N G S P R O T O K O L L

 OSKARSHAMN II ERS2
 PATRON NR 3058 DATUM: 12-04-77 RFF.
 Rev. 80-12-15

*	2.80	2.02	2.02	2.02	2.02	2.80	2.80	1.18
*	2.997	2.968	3.304	3.303	3.305	3.300	3.003	3.303
8*	6301	5815	20493	20557	20535	22014	6194	20223
*								
*	2.80	2.80	2.80	2.80	3.503A	2.80	2.02	2.80
*	3.303	3.306	3.300	3.310	3.256	3.302	3.304	3.004
7*	21996	21816	21820	327	15328	22012	20642	6311
*								
*	2.80	3.503A	3.50		2.80	3.50	2.80	2.80
*	3.302	3.254	3.292		3.310	3.291	3.301	3.297
6*	22000	15261	23695		21801	23813	21824	21967
*								
*	3.50	3.50		1.18		2.80	3.503A	2.02
*	3.292	3.305		3.306		3.309	3.252	3.307
5*	23750	47		20200		21907	15376	20641
*								
*	3.50	3.50	3.50		1.18		2.80	2.02
*	3.289	3.290	3.298		3.303		3.303	3.303
4*	23692	23817	23794	21223	20172		417	20674
*								
*	3.50		3.50	3.50		3.50	2.80	2.02
*	3.292		3.285	3.292		3.295	3.259	3.307
3*	23791		23819	23321		23738	21973	20673
*								
*	3.50	3.503A		3.50	3.50	3.503A	2.80	2.02
*	3.299	3.251		3.298	3.283	3.252	3.305	2.976
2*	23772	15373		23736	179	15372	21854	5852
*								
*	2.80	3.50	3.50	3.50	3.50	2.80	2.80	2.80
*	3.005	3.290	3.288	3.299	3.293	3.302	3.303	3.001
1*	6244	23805	23806	23762	23761	21813	21864	6248

A	B	C	D	E	F	G	H
ANRIKN.%	ANTAL	KG	KG	KG			
NOM	VEPKL	STAVAR	U02	U	U-235		
1.18	1.204	3	9.912	28.732	0.105		
2.02	2.031	9	29.082	25.621	0.520		
2.80	2.810	21	67.862	59.736	1.680		
3.50	3.527	18	59.271	52.218	1.842		
3.503A	3.492	5	16.265	14.014	0.489		
S=A	2.765	56	182.392	160.372	4.637	TOPPL.NP.: 22244	90TTP.NP.: 2792
SPRIDARE NP.:		7963	7966	7996	8106	7938	8361
BOX NR.:	3852						
URSPR.LAND FÖR URAN:	USA	INBLANDAD MÄNGD	GD2	03=4.0%			

Oskarshamn 2 Fuel rod diagram for assembly 3064

M O N T E R I N G O P P O T O K O L L

 OSKARSHAMN II EPS2
 DATUM:12-04-77

PATRON NR 3064 REF. NR

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*****
* 2.02 2.02 2.02 2.02 2.02 2.02 1.18 1.18
* 2.968 2.966 3.305 3.312 3.303 3.307 2.993 2.971
8* 5754 5768 20478 20662 20612 20507 5369 5360
*
* 2.80 2.80 2.80 2.80 3.50BA 2.80 2.02 1.18
* 2.998 3.303 3.307 3.307 3.242 3.296 3.298 2.986
7* 6247 21337 21994 269 15433 21846 20565 5363
*
* 2.80 3.50BA 3.50 2.80 2.80 3.50 2.80 2.02
* 3.306 3.246 3.290 3.300 3.302 3.295 3.298 3.306
6* 21835 15435 23727 21992 22018 23623 21968 20564
*
* 3.50 3.50 2.80 1.18 1.18 2.80 3.50PA 2.02
* 3.287 3.299 3.303 3.298 3.301 3.294 3.248 3.306
5* 23726 69 21861 20187 20205 21833 15449 20616
*
* 3.50 3.50 3.50 1.18 2.80 2.80 2.80 2.02
* 3.293 3.295 3.296 3.302 3.305 3.306 3.300 3.300
4* 23675 23659 23678 20085 20159 21860 270 20665
*
* 3.50 3.50 3.50 3.50 2.80 3.50 2.80 2.02
* 3.293 3.294 3.290 3.292 3.304 3.290 3.297 3.308
3* 23654 23686 23640 23612 22011 23724 21951 20517
*
* 2.80 3.50BA 3.50 3.50 3.50 3.50BA 2.80 2.02
* 2.999 3.258 3.298 3.292 3.298 3.247 3.304 2.979
2* 6148 15311 23666 23744 92 15439 22001 5825
*
* 2.80 2.80 3.50 3.50 3.50 2.80 2.80 2.02
* 3.002 2.996 3.288 3.295 3.293 3.294 3.007 2.967
1* 6337 6213 23751 23646 23749 22010 6178 5802
*****
A B C D E F G H
ANRIKN.% ANTAL KG KG KG
NOM VERKL STAVAR UO2 U U-235
1.18 1.204 6 18.851 16.608 0.200
2.02 2.031 13 41.630 36.676 0.745
2.80 2.809 21 67.828 59.756 1.679
3.50 3.527 18 59.278 52.224 1.842
3.50BA 3.492 5 16.241 13.993 0.489
S:A 2.765 63 203.828 179.257 4.955
SPRIDARE NR.: 8383 8398 8393 8376 8259 9324
BOX NR.:4313
URSPR.LAND FÖR URAN: USA INBLANDAD MÄNGD GD2 O3=4.0%
  
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Oskarshamn 2 Fuel rod diagram for assembly 3064

M O N T E R I N G S P R O T O K O L L

 OSKARSHAMN II ERS2
 PATRON NR 3764 DATUM: 12-14-77 REF. NR

Rev 80-12-17

* 2.80	2.02	2.02	2.02	2.02	2.80	2.30	1.18
* 2.998	2.966	3.305	3.312	3.303	3.300	2.999	3.301
8* 6247	5768	20478	20662	20612	21992	6148	20205

* 2.80	2.80	2.80	2.80	3.50BA	2.80	2.02	2.80
* 3.303	3.303	3.307	3.307	3.242	3.296	3.298	2.996
7* 21861	21337	21994	269	15433	21846	20565	6213

* 2.80	3.50BA	3.50		2.80	3.50	2.80	2.80
* 3.306	3.246	3.290		3.302	3.295	3.298	3.305
6* 21835	15435	23727		22018	23623	21968	21860

* 3.50	3.50		1.18		2.80	3.50BA	2.02
* 3.297	3.299		3.298		3.294	3.248	3.306
5* 23726	69		20187		21833	15449	20616

* 3.50	3.50	3.50		1.18		2.80	2.02
* 3.293	3.295	3.296		3.302		3.306	3.300
4* 23675	23659	23678	20085	20159		270	20665

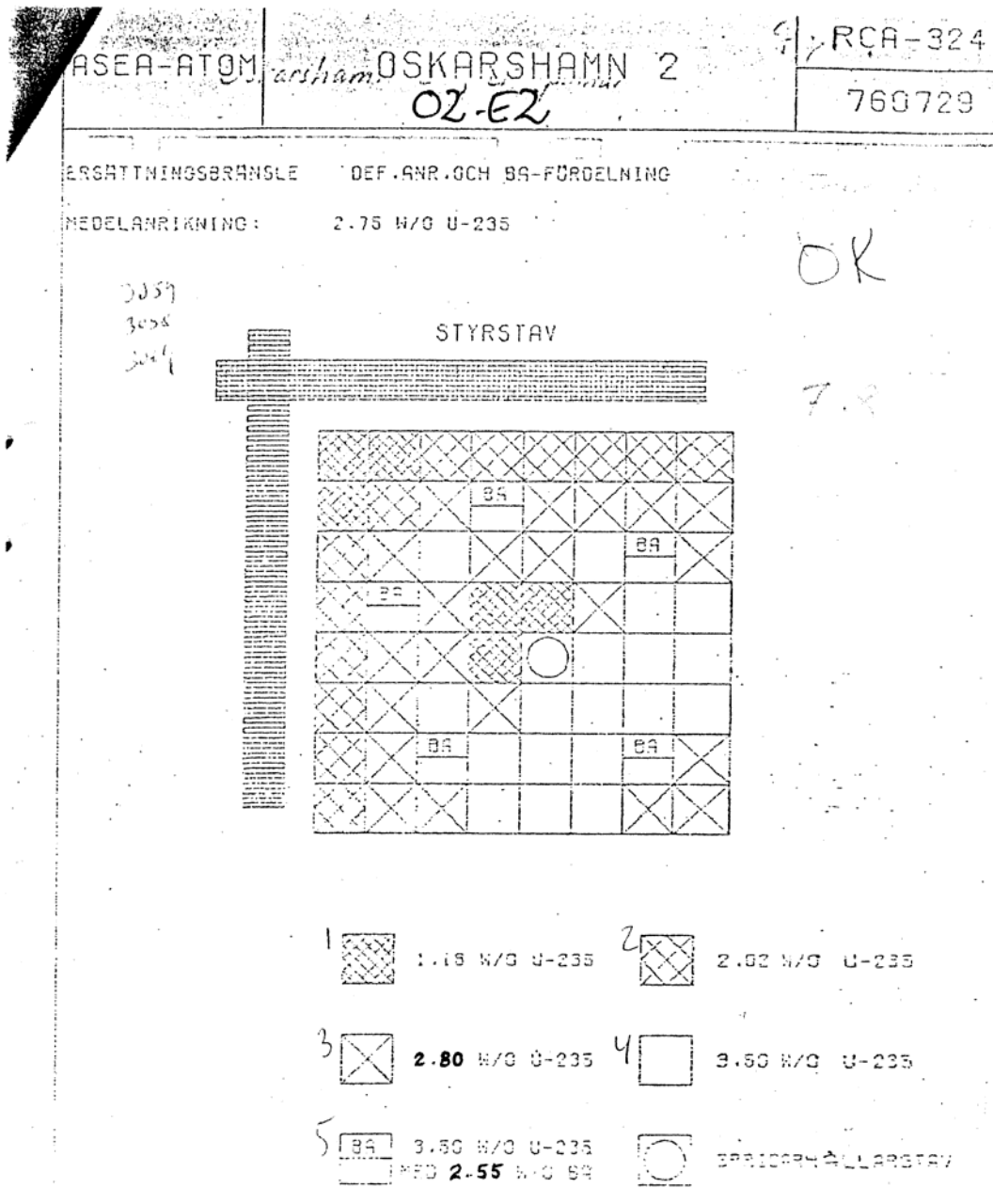
* 3.50		3.50	3.50		3.50	2.80	2.02
* 3.293		3.290	3.292		3.290	3.297	3.308
3* 23654		23640	23612		23724	21951	20517

* 3.50	3.50BA		3.50	3.50	3.50BA	2.80	2.02
* 3.294	3.258		3.292	3.298	3.247	3.304	2.979
2* 23686	15311		23744	92	15439	22001	5825

* 2.80	3.50	3.50	3.50	3.50	2.80	2.80	2.80
* 3.002	3.298	3.288	3.295	3.293	3.294	3.304	3.007
1* 6337	23666	23751	23646	23749	22010	22011	6178

	A	B	C	D	E	F	G
							H
ANRIKN.%	ANTAL	KG	KG	KG			
NOM VERKL STAVAR		U02	U	U-235			
1.18 1.204	3	9.901	2.723	0.105			
2.02 2.031	9	29.082	25.621	0.520			
2.80 2.809	21	67.828	59.756	1.679			
3.50 3.527	18	59.278	52.224	1.842			
3.50BA 3.492	5	16.241	13.993	0.489			
S:A 2.765	56	182.330	160.318	4.635			
SPRIDARE NR.:		8383	8398	8393	8376	8259	9324
BOX NR.:		4313					
URSPR.LAND FÖR URAN:		USA	INBLANDAD	MÄNGD	GD2	03=4.0%	
						TOPPL.NR. : 21991	
						BOTTPL.NR. : 2729	

Oskarshamn 2 Enrichment diagram for assembly 3054, 3058 och 3064



OSKARSHAMN 2

FIGUR 26

OSKARSHAMN 2 RELOAD FUEL E3

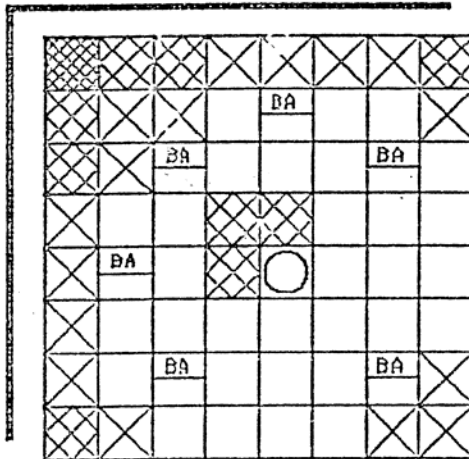
AVERAGE ENRICHMENT: 2.75W/O U-235






7.10

6350

OK

CONTROL ROD



-  1.38 W/O U-235
-  1.82 W/O U-235
-  2.34 W/O U-235
-  3.17 W/O U-235
-  SPACER CAPTURE ROD

3.17 W/O U-235
WITH 3.2 H/O BA

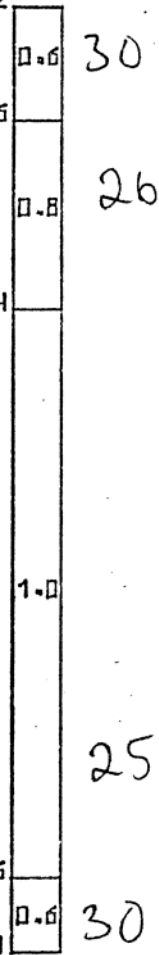
3712

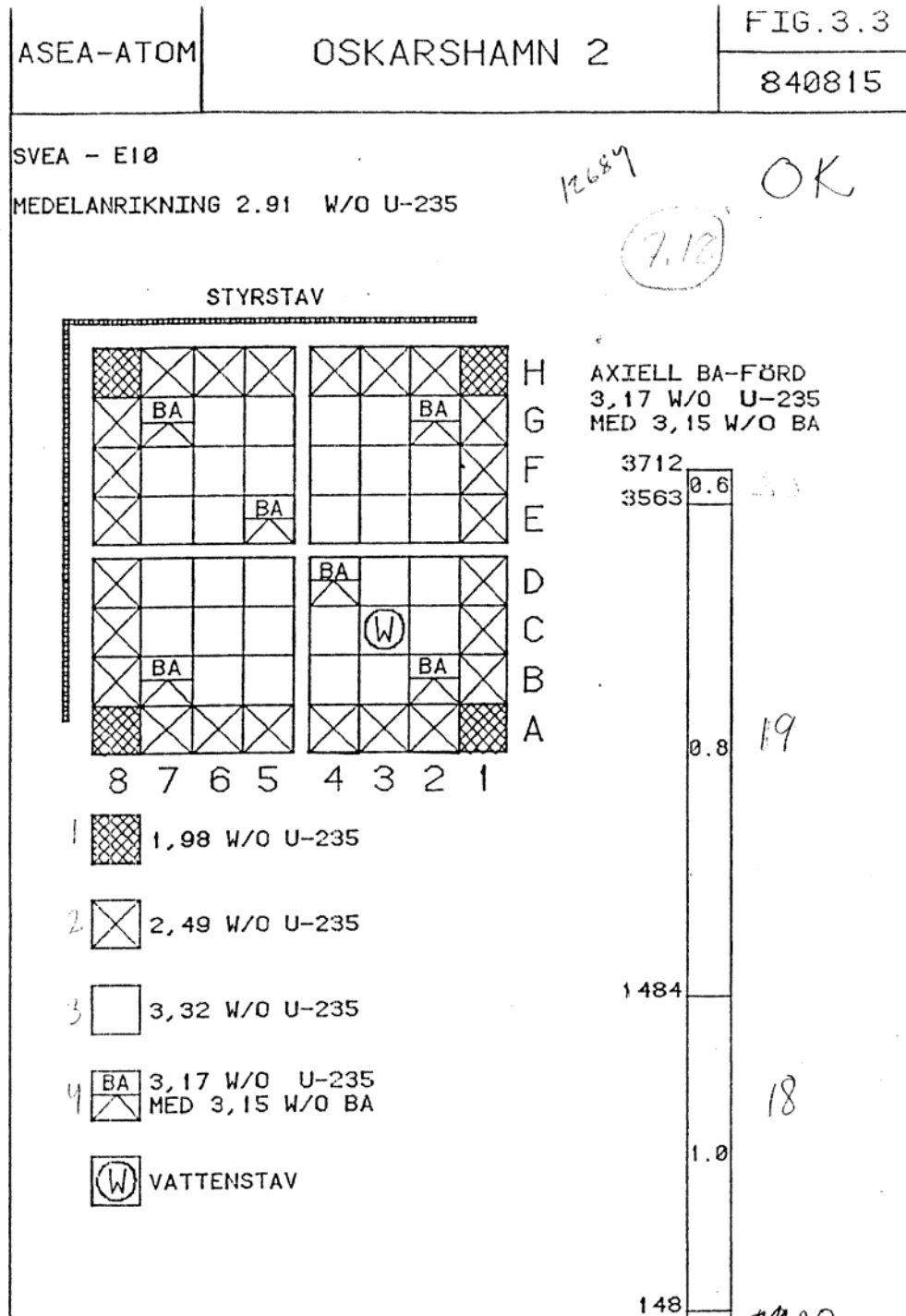
3266

2524

296

0





2.5 Oskarshamn 3

REACTOR OSKARSHAMN 3

Reactor data	Value
Reactor pressure (MPa) at full power	7
Inlet temperature into core (C) at full power	277
Outlet temperature into core (C) at full power	286
Avg. temperature in the uranium pellets at full power (C)	507
Avg. temperature in the cladding at full power (C)	290
Temperature in the boxwall at full power (C)	286

Cycle data

Cycle no	Start	Stop	Reactor power MW	Burnup avg/cycle MWd/tU	No of assemblies in core
1	1985-03-18	1986-07-04	3,300	7,939.9	700
2	1986-07-26	1987-07-03	3,300	7,335.7	700
3	1987-07-26	1988-07-08	3,300	7,174	700
4	1988-08-14	1988-12-10	3,300	2,211	700
4B	1988-12-22	1989-06-07	3,300	3,199.7	700
5	1989-06-20	1990-06-23	3,300	7,595	700
6	1990-08-01	1991-06-24	3,300	7,718.6	700

Fuel data

	Fuel id 12078	13628	13630
Fuel type	8x8	Svea100	Svea100
No of fuel rods	63	100	100
Rod pitch normal rods (mm)	16.3	12.4	12.4
Rod pitch normal – corner rods (mm)	16.05		
Rod pitch corner – corner rods (mm)	15.8		
Rod diameter normal rod (mm)	12.25	9.62	9.62
Clad thickness normal rod (mm)	0.8	0.63	0.63
Pellet diameter normal rod (mm)	10.44	8.19	8.19
No of normal rods	51	100	100
Rod diameter corner rod (mm)	11.75		
Clad thickness corner rod (mm)	0.8		
Pellet diameter corner rod (mm)	9.94		
No of corner rods	12		
Active length (mm)	3680	3750	3750
Density UO ₂ (g/cc)	10.438	10.497	10.497
Porsity,dishing etc (%)	1	1	1
No of spacer rods		4	4
Material in spacer rods		Zr2	Zr2
Outer diameter spacer rods (mm)		9.62	9.62
Cladding thickness spacer rod		0.63	0.63
No of water rods	1 sprh		
Material in water rods	Zry		
Outer diameter water rods (mm)	12.25		
Inner diameter water rods (mm)	10.65		
No of BA rods	3	8	8
% Gd ₂ O ₃	5.5	4.4	4.4
Rel poison	1	1	1
No of spacers	6	4x6	4x6
Spacer material	Inconel	Inconel	Inconel
Mass of spacer (g)	150	24	24
Box material	Zr4	Zr4	Zr4
Box outer measure square (mm)	138.6	139.6	139.6
Box wall thickness (mm)	2.3	1,1/0,8	1,1/0,8

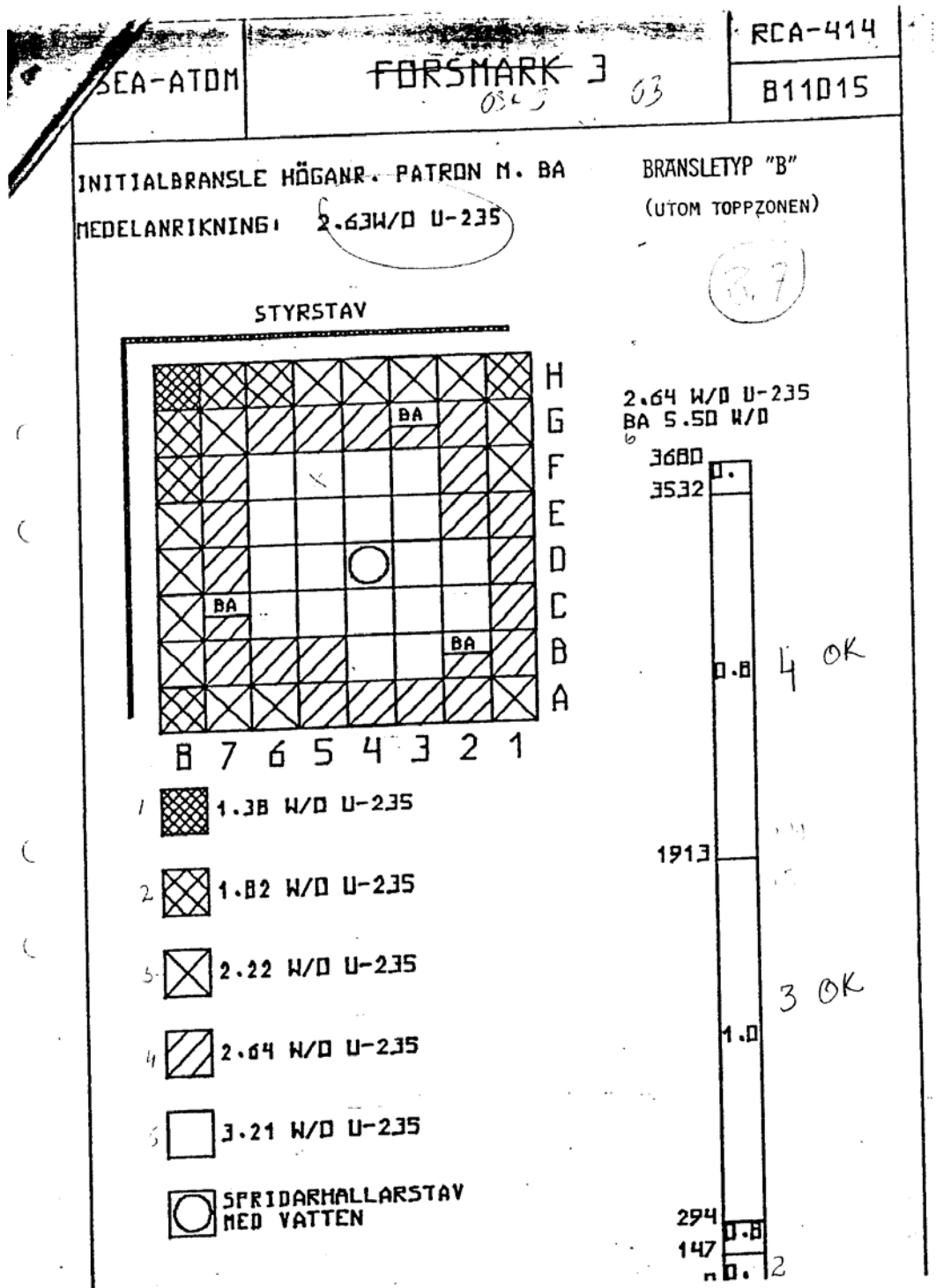
Initial data	12078	13628	13630
Initial mass Utot (g)	177,358	180,774	180,775
Initial mass U238 (g)	172,787	175,873	175,874
Initial mass U235 (g)	4,571	4,901	4,901
Avg.enrichment% U235	2.577	2.711	2.711
Data after rebuild 1			
Date of rebuild			
No of fuel rods			
No spacer rods			
No water rods			
No of water holes			
Mass Utot after rebuild (g)			
Mass U235 after rebuild (g)			
Data after rebuild 2			
Date of rebuild			
No of fuel rods			
No spacer rods			
No water rods			
No of water holes			
Mass Utot after rebuild (g)			
Mass U235 after rebuild (g)			
Cycle history burnup/cycle, MWd/tU			
1	8,690		
2	8,258	9,171	9,328
3	8,212	9,442	9,656
4		2,297	2,986
4B		3,763	3,823
5		8,212	7,073
6		2,734	7,497
Axial BU distribution, EOL MWd/tUNode no (top=25)			
1	11,300	7,800	9,400
2	19,500	27,300	31,800
3	23,600	34,100	39,200
4	25,400	37,000	42,300
5	27,000	39,100	44,700
6	27,700	40,100	45,700
7	27,900	40,400	46,200
8	27,700	40,200	46,000
9	28,300	40,900	46,700
10	28,600	41,200	47,000
11	28,600	41,200	46,900
12	28,300	40,700	46,300
13	28,900	41,100	46,700
14	29,300	41,100	46,600
15	29,200	40,900	46,200
16	28,700	40,400	45,500
17	28,900	40,800	45,700
18	28,600	40,600	45,300
19	27,900	40,100	44,500
20	26,600	38,700	42,900
21	25,700	37,500	41,500
22	23,900	34,800	38,700
23	21,000	30,500	34,100
24	16,700	24,100	27,700
25	9,400	9,800	11,500

Void history	12078	13628	13630
Node no (top=25)			
1	-0.03	-0.02	-0.02
2	-0.02	-0.02	-0.02
3	0.01	0.00	0.00
4	0.07	0.06	0.06
5	0.14	0.15	0.15
6	0.21	0.22	0.23
7	0.28	0.29	0.30
8	0.34	0.35	0.36
9	0.39	0.40	0.41
10	0.44	0.45	0.46
11	0.48	0.49	0.51
12	0.51	0.52	0.54
13	0.55	0.55	0.58
14	0.58	0.58	0.61
15	0.61	0.61	0.63
16	0.63	0.63	0.66
17	0.65	0.65	0.68
18	0.67	0.67	0.70
19	0.69	0.69	0.71
20	0.71	0.70	0.73
21	0.72	0.72	0.74
22	0.73	0.73	0.75
23	0.74	0.74	0.76
24	0.75	0.74	0.76
25	0.75	0.74	0.76

Density history
Node no (top=25)
1
2
3
4
5
6
7
8
9
10
11
12
13
14
15
16
17
18
19
20
21
22
23
24
25

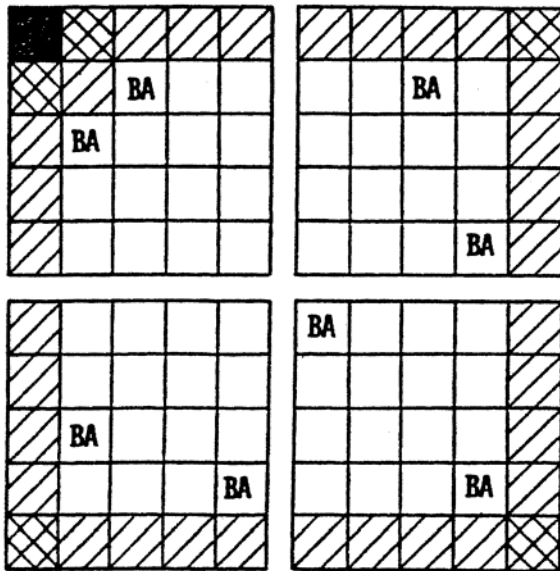
Control rod history
Node no (top=25)

1	0.00	0.16	0.02
2	0.00	0.09	0.01
3	0.00	0.07	0.00
4	0.00	0.05	0.00
5	0.00	0.05	0.00
6	0.00	0.05	0.00
7	0.00	0.05	0.00
8	0.00	0.05	0.00
9	0.00	0.05	0.00
10	0.00	0.05	0.00
11	0.00	0.04	0.00
12	0.00	0.04	0.00
13	0.00	0.04	0.00
14	0.00	0.04	0.00
15	0.00	0.04	0.00
16	0.00	0.03	0.00
17	0.00	0.03	0.00
18	0.00	0.02	0.00
19	0.00	0.01	0.00
20	0.00	0.00	0.00
21	0.00	0.00	0.00
22	0.00	0.01	0.00
23	0.00	0.01	0.00
24	0.00	0.01	0.00
25	0.00	0.00	0.00



DEMO 5 x 5. ANRIKNINGSFÖRDELNING
OCH BA-DIMENSIONERING

DEMO-86 MM








3750
3600

NAT
U

19
UTAN BA

~~13628~~
13628
13630

18

- 1  1.71 W/O U-235
- 2  1.98 W/O U-235
- 3  2.49 W/O U-235
- 4  3.17 W/O U-235
- 5  3.17 W/O U-235 MED 4.4 W/O Gd₂O₃

3.13

HOG
ANR
ZON

MED BA

MEDELANRIKNING I HOGANRIKAD ZON
= 2.881 W/O U-235

2.6 Ringhals 1

REACTOR RINGHALS 1

Reactor data	Value
Reactor pressure (MPa) at full power	7
Inlet temperature into core (C) at full power	272
Outlet temperature into core (C) at full power	286
Avg. temperature in the uranium pellets at full power (C)	615
Avg. temperature in the cladding at full power (C)	295
Temperature in the boxwall at full power (C)	290

Cycle data

Cycle no	Start	Stop	Reactor power MW	Burnup avg/cycle MWd/tU	No of assemblies in core
C1A	1973-08-20	1976-05-22	2,270	3,220	648
C1A	1976-09-28	1977-07-27	2,270	3,626	648
C1B	1977-11-04	1978-07-13	2,270	4,487	648
C2	1978-10-13	1979-07-12	2,270	4,280	648
C3	1979-09-30	1980-07-03	2,270	4,559	648
C4	1980-10-08	1981-07-30	2,270	5,260	648
C5	1981-11-06	1982-07-22	2,270	4,438	648
C6	1982-09-09	1983-06-18	2,270	4,490	648
C7	1983-09-30	1984-07-13	2,270	4,533	648
C8	1984-08-28	1985-08-02	2,270	5,984	648
C9	1985-09-02	1986-08-15	2,270	6,018	648
C10	1986-10-09	1987-08-21	2,270	5,508	648
C11	1987-09-24	1988-08-06	2,270	5,529	648
C12	1988-09-11	1989-09-15	2,384	5,832	648
C13	1989-10-11	1990-08-04	2,500	5,030	648
C14	1990-08-30	1991-08-09	2,500	5,566	648
C15	1991-09-06	1992-07-25	2,500	6,212	648
C16	1992-08-22	1993-10-01	2,500	3,956	648

Initial data	582	596	710	900	1136	1177	1186	5829	6423	6432	6454	6478	8327	8331	8332	8338	8341
Initial mass Utot (g)	180,265	180,057	180,181	180,321	180,016	180,587	180,515	170,200	177,701	177,520	177,683	177,568	177,544	177,690	177,519	177,596	176,475
Initial mass U238 (g)	176,193	175,991	176,107	176,244	175,942	175,816	175,750	165,591	172,547	172,382	172,534	172,419	172,389	172,520	172,380	172,426	171,376
Initial mass U235 (g)	4,072	4,066	4,074	4,077	4,074	4,771	4,765	4,609	5,154	5,138	5,149	5,149	5,155	5,170	5,139	5,170	5,099
Avg.enrichment% U235	2.259	2.258	2.261	2.261	2.263	2.642	2.640	2.708	2.900	2.894	2.898	2.900	2.904	2.910	2.895	2.911	2.889
Data after rebuild 1																	
Date of rebuild	1978-08-23	1978-09-01	1978-09-11	1978-08-22	1978-09-06				81-10-14				1988-01-21				
No of fuel rods	63	63	63	63	63				62				44				
No spacer rods	zr homo	zr homo	zr homo	zr homo	zr homo												
No water rods																	
No of water holes																	
Mass Utot after rebuild (g)	177,394	177,199	177,308	177,362	177,162				167,335				126,675				
Mass U235 after rebuild (g)	4,000	3,995	4,003	4,004	4,003				4,557				3,673				
Data after rebuild 2																	
Date of rebuild						1981-09-14			87-09-22								
No of fuel rods						60			58								
No spacer rods																	
No water rods																	
No of water holes																	
Mass Utot after rebuild (g)						171,581			156,410								
Mass U235 after rebuild (g)						4,274			4,294								
Cycle history burnup/cycle, MWd/tU																	
C1A	3,300	3,100	3,300	3,600	3,500												
C1A	3,970	4,100	3,660	4,020	3,940												
C1B	5,013	5,305	4,747	4,528	4,178												
C2	3,972	4,777	4,630	4,035	4,956	4,486	4,554										
C3	5,015	4,974	5,054	4,662		4,883	5,060	6,254									
C4						6,965	5,845	7,354			7,120	6,709					
C5			1,223	2,307	2,472	4,096	4,508	5,478		5,502	5,880	5,678			5,755	5,660	5,358
C6					3,184	5,068	4,625	4,154	5,401	4,942	5,936	4,324			5,713	5,470	5,245
C7						4,822	3,428	4,669	6,663	5,737	5,895	5,421	3,212	6,974	5,090	5,152	5,213
C8						5,922	2,478	5,826	7,376	7,085	6,087	6,934	6,270	8,158	6,849	6,738	6,082
C9								5,864	6,749	6,813	6,318	6,117	6,474	7,361	6,486	6,943	4,508
C10								5,262	5,446	4,667			5,622	4,155	2,758	2,671	6,392
C11									3,474	2,115			6,487	6,671	2,326	2,196	1,301
C12													6,755	2,584			
C13													1,321				
C14													1,710				

Axial BU distribution, EOL MWd/t UNode no (top=25)	582	596	710	900	1136	1177	1186	5829	6423	6432	6454	6478	8327	8331	8332	8338	8341
1						-0.01	0.00	-0.01									
2						0.00	0.00	0.00									
3						0.02	0.02	0.02									
4						0.05	0.06	0.06									
5						0.10	0.11	0.10									
6						0.16	0.15	0.15									
7						0.22	0.20	0.21									
8						0.27	0.25	0.26									
9						0.32	0.30	0.31									
10						0.37	0.34	0.35									
11						0.42	0.37	0.40									
12						0.45	0.41	0.43									
13						0.49	0.43	0.47									
14						0.52	0.46	0.50									
15						0.54	0.49	0.53									
16						0.57	0.51	0.55									
17						0.59	0.53	0.58									
18						0.61	0.55	0.60									
19						0.63	0.57	0.61									
20						0.65	0.59	0.63									
21						0.66	0.60	0.65									
22						0.67	0.62	0.66									
23						0.68	0.63	0.67									
24						0.69	0.63	0.68									
25						0.70	0.64	0.68									

Void history
Node no (top=25)

- 1
- 2
- 3
- 4
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- 20

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Control rod history
Node no (top=25)

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Ringhals 1 Fuel rod diagram for assembly 582

MONTFRINGS PROTOKOLL

PATRON NR. 582.

DATUM: 05-01-73

REF. HÖRN

* 1.85	1.85	1.85	1.85	1.85	1.40	1.46	1.40
* 2.950	2.960	3.252	3.232	3.253	3.242	2.932	2.942
* 5230	5233	19219	19169	19231	18002	6419	6442
* 2.50	2.50	2.50	2.50	1.85	1.85	1.85	1.40
* 2.944	3.253	3.253	3.241	3.230	3.248	3.256	2.949
* 11571	24298	24286	24244	19149	647	19196	6401
* 2.50	2.50	2.50	2.50	2.50	2.50	1.85	1.40
* 3.266	3.254	3.250	3.265	3.264	3.259	3.246	3.230
* 24445	1493	24246	24426	24407	24302	19096	18009
* 2.50	2.50	2.50	2.50	2.50	2.50	1.85	1.85
* 3.264	3.256	3.259	3.252	3.267	3.265	3.249	3.228
* 24283	24425	24285	24249	24314	24326	19103	19125
* 2.50	2.50	2.50	2.50	2.50	2.50	2.50	1.85
* 3.260	3.258	3.259	3.259	3.271	3.268	3.269	3.250
* 24257	24313	24363	24237	24339	24393	24317	19232
* 2.50	2.50	2.50	2.50	2.50	2.50	2.50	1.85
* 3.253	3.267	3.254	3.257	3.263	3.241	3.251	3.216
* 24321	24263	24394	24344	24353	24241	1448	19161
* 2.50	2.50	2.50	2.50	2.50	2.50	2.50	1.85
* 2.954	3.255	3.252	3.269	3.257	3.248	3.261	2.957
* 11577	24398	1572	24360	24294	24307	24350	5556
* 1.85	2.50	2.50	2.50	2.50	2.50	2.50	1.85
* 2.952	2.945	3.262	3.261	3.266	3.261	2.948	2.959
* 5264	11567	24247	24320	24351	24417	11532	5475
ANRIKN.%	ANTAL	KG	KG	KG			
NOM VERKL.	STAVAR	U-2	U	U-235			
1.40	1.412	5	15.295	13.475	0.190	TOPPL.NR.:4065A	
1.05	1.853	16	50.436	44.436	0.824		
2.50	2.499	43	138.881	122.354	3.058	BOTTPL.NR.:3892B	
S:A	2.259	64	204.614	180.265	4.072		

SPRIDARE NR.: 2075 2558 904 2931 2059 2526 UPSPR.LAND FÖR URAN: US,
 BOXNR.: 1150

Ringhals 1 Fuel rod diagram for assembly 582

MONITERINGS PROTOKOLL

PATRON NR. 582. BATH: 05-01-73 REF. 0000
 SPRIDARHÄLLARSTAV 78-08-23

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*****
* 1.85 1.85 1.85 1.85 1.85 1.40 1.40 1.40 *
* 2.950 2.960 3.252 3.232 3.253 3.242 2.932 2.942 *
* 5230 5233 19219 19169 19231 18002 6419 6442 *
*
* 2.50 2.50 2.50 2.50 1.85 1.85 1.85 1.40 *
* 2.944 3.253 3.253 3.241 3.230 3.248 3.256 2.949 *
* 11571 24298 24286 24244 19149 647 19196 6401 *
*
* 2.50 2.50 2.50 2.50 2.50 2.50 1.85 1.40 *
* 3.266 3.254 3.250 3.265 3.264 3.259 3.246 3.230 *
* 24446 1493 24246 24426 24407 24302 19096 18009 *
*
* 2.50 2.50 2.50 2.50 2.50 2.50 1.85 1.85 *
* 3.264 3.256 3.259 3.252 3.267 3.265 3.249 3.228 *
* 24283 24425 24285 24249 24314 24326 19103 19125 *
*
* 2.50 2.50 2.50 2.50 2.50 2.50 2.50 1.85 *
* 3.260 3.258 3.259 3.271 3.268 3.269 3.250 3.250 *
* 24257 24313 24363 24339 24393 24317 19232 *
*
* 2.50 2.50 2.50 2.50 2.50 2.50 2.50 1.85 *
* 3.253 3.267 3.254 3.257 3.263 3.241 3.251 3.216 *
* 24321 24263 24394 24344 24353 24241 1448 19161 *
*
* 2.50 2.50 2.50 2.50 2.50 2.50 2.50 1.85 *
* 2.954 3.255 3.252 3.269 3.257 3.248 3.261 2.957 *
* 11577 24398 1572 24360 24294 24387 24350 5556 *
*
* 1.85 2.50 2.50 2.50 2.50 2.50 2.50 1.85 *
* 2.952 2.945 3.262 3.261 3.266 3.261 2.946 2.959 *
* 5264 11567 24247 24320 24351 24417 11532 5475 *
    
```

```

*****
ANRIKN.% ANTAL KG KG KG
NON VERKL. STAVAR UG2 U U-235
1.40 1.412 5 15.295 13.475 0.190 TOPPL.NR.:4065A
1.85 1.853 16 50.430 44.156 0.824
2.50 2.499 42 135.622 119.483 2.986 BOTTPL.NR.:3892H
S:A 2.255 63 201.355 177.394 4.000
    
```

SPRIDARE NR.: 2075 2050 204 2011 2052 2026 UPSPP.LAND FÖR URAN: US1
 SÖXNR.: 1108

Ringhals 1 Fuel rod diagram for assembly 596

MONTERINGSPROTOKOLL

PATRON NR. 596.

DATUM:12-01-73

REF.HÖRN

* 1.85	1.85	1.85	1.85	1.85	1.40	1.40	1.40	*
* 2.951	2.951	3.238	3.236	3.234	3.250	2.956	2.952	*
* 5403	5428	19186	19095	19065	18146	6399	6390	*
* 2.50	2.50	2.50	2.50	1.85	1.85	1.85	1.40	*
* 2.933	3.245	3.257	3.258	3.245	3.239	3.234	2.960	*
* 11615	25465	25497	25446	19164	471	19060	6368	*
* 2.50	2.50	2.50	2.50	2.50	2.50	1.85	1.40	*
* 3.250	3.247	3.254	3.247	3.261	3.248	3.226	3.252	*
* 25564	2000	25561	25565	25422	25556	19072	18060	*
* 2.50	2.50	2.50	2.50	2.50	2.50	1.85	1.85	*
* 3.246	3.250	3.248	3.251	3.254	3.256	3.238	3.245	*
* 25529	25566	25541	25571	25551	25552	19133	19132	*
* 2.50	2.50	2.50	2.50	2.50	2.50	2.50	1.85	*
* 3.256	3.254	3.258	3.244	3.256	3.256	3.259	3.227	*
* 25559	25474	25523	25563	25554	25506	25555	19121	*
* 2.50	2.50	2.50	2.50	2.50	2.50	2.50	1.85	*
* 3.254	3.255	3.252	3.263	3.257	3.253	3.256	3.245	*
* 25488	25542	25560	25360	25568	25558	1982	19097	*
* 2.50	2.50	2.50	2.50	2.50	2.50	2.50	1.85	*
* 2.948	3.256	3.261	3.247	3.251	3.256	3.259	2.959	*
* 11676	25451	1954	25414	25487	25409	25483	5385	*
* 1.85	2.50	2.50	2.50	2.50	2.50	2.50	1.85	*
* 2.957	2.940	3.246	3.255	3.251	3.247	2.930	2.957	*
* 5351	11684	23436	25557	25435	25518	11742	5382	*

ANRIKN.%	ANTAL	KG	KG	KG	
NOM VERKL.	STAVAR	UO2	U	U-235	
1.40	1.412	5	15.370	13.541	0.191
1.85	1.854	16	50.382	44.387	0.823
2.50	2.499	43	138.625	122.129	3.052
S:A	2.258	64	204.377	180.057	4.066

TOPPL.NR.:3872H
BOTTPL.NR.:3199C

SPRIDARE NR.: 2592 2596 2581 2249 2496 2274 UPSPP-LAND FÖR HRAN: USA
BOXNR.: 522

Ringhals 1 Fuel rod diagram for assembly 596

 MONTERINGS PROTOKOLL

PATRON NR. 596.

DATUM:12-01-73

REF.HÖRN

SPRIDARHÄLLARSTAV 780901

* 1.85	1.85	1.85	1.85	1.85	1.40	1.40	1.40	*
* 2.951	2.951	3.238	3.236	3.234	3.250	2.956	2.952	*
* 5403	5428	19186	19095	19065	18146	6399	6390	*
* 2.50	2.50	2.50	2.50	1.85	1.85	1.85	1.40	*
* 2.933	3.245	3.257	3.258	3.245	3.239	3.234	2.960	*
* 11615	25465	25497	25446	19164	471	19060	6368	*
* 2.50	2.50	2.50	2.50	2.50	2.50	1.85	1.40	*
* 3.250	3.247	3.254	3.247	3.261	3.248	3.226	3.252	*
* 25564	2000	25561	25565	25422	25556	19072	18060	*
* 2.50	2.50	2.50	2.50	2.50	2.50	1.85	1.85	*
* 3.246	3.250	3.248	3.251	3.254	3.256	3.238	3.245	*
* 25529	25566	25541	25571	25551	25552	19133	19132	*
* 2.50	2.50	2.50	2.50	2.50	2.50	2.50	1.85	*
* 3.256	3.254	3.258	3.256	3.256	3.256	3.259	3.227	*
* 25559	25474	25523	25554	25506	25555	19121	19121	*
* 2.50	2.50	2.50	2.50	2.50	2.50	2.50	1.85	*
* 3.254	3.255	3.252	3.263	3.257	3.253	3.256	3.245	*
* 25488	25542	25560	25360	25568	25558	1982	19097	*
* 2.50	2.50	2.50	2.50	2.50	2.50	2.50	1.85	*
* 2.948	3.256	3.261	3.247	3.251	3.256	3.259	2.959	*
* 11676	25451	1954	25414	25487	25409	25483	5385	*
* 1.85	2.50	2.50	2.50	2.50	2.50	2.50	1.85	*
* 2.957	2.940	3.246	3.255	3.251	3.247	2.930	2.957	*
* 5351	11684	23436	25557	25435	25518	11742	5382	*

ANRIKN.%	ANTAL	KG	KG	KG	
NOM	VERKL.	UO2	U	U-235	
1.40	1.412	5	15.370	13.541	0.191
1.85	1.854	16	50.382	44.387	0.823
2.50	2.499	42	135.381	119.271	2.981
S:A	2.254	63	201.133	177.199	3.995

SPRIDARE NR.: 2592 2596 2581 2249 2406 2274 UO2SP.LAND FÖR URÄN: USA
 BOXNR.: 522

Ringhals 1 Fuel rod diagram for assembly 710

PATRON NR. 710.

DATUM: 12-02-73

REF. HÖRNI

1.85	1.85	1.85	1.85	1.85	1.40	1.40	1.40
2.941	2.955	3.251	3.259	3.255	3.236	2.951	2.943
9117	9217	27485	27421	27420	21438	7116	7032
2.50	2.50	2.50	2.50	1.85	1.85	1.85	1.40
2.941	3.253	3.247	3.255	3.264	3.241	3.267	2.941
11555	31649	31629	31429	27474	479	27483	7163
2.50	2.50	2.50	2.50	2.50	2.50	1.85	1.40
3.257	3.242	3.259	3.245	3.247	3.250	3.253	3.255
31641	1610	31634	31492	31643	31518	27501	21498
2.50	2.50	2.50	2.50	2.50	2.50	1.85	1.85
3.259	3.253	3.250	3.256	3.246	3.257	3.258	3.261
31603	31556	31540	31642	31648	31646	27430	27504
2.50	2.50	2.50	2.50	2.50	2.50	2.50	1.85
3.256	3.250	3.260	3.261	3.253	3.251	3.255	3.255
31572	31609	31537	31578	31615	31588	31613	27449
2.50	2.50	2.50	2.50	2.50	2.50	2.50	1.85
3.255	3.248	3.251	3.262	3.244	3.260	3.257	3.256
31536	31597	31636	31520	31539	31577	1693	27464
2.50	2.50	2.50	2.50	2.50	2.50	2.50	1.85
2.948	3.261	3.252	3.251	3.261	3.250	3.254	2.942
11574	31631	1600	31529	31527	31517	31535	9170
1.85	2.50	2.50	2.50	2.50	2.50	2.50	1.85
2.945	2.933	3.247	3.249	3.255	3.260	2.943	2.946
9268	11044	31426	31530	31614	31617	11073	9241

ANRIKVN. %	ANTAL	KG	KG	KG	
NOM VERKI.	SIAMAR	U-2	U	U-235	
1.40	1.412	5	15.326	13.502	0.191
1.85	1.853	16	50.549	44.534	0.825
2.50	2.504	43	138.644	122.145	3.058
S:A	2.261	64	204.519	180.181	4.074

SPRIDARE NR.: 3180 3258 3200 3106 3265 3182 URSPR. LAND FÖR URAN: US
 BOXNR.: 662

Ringhals 1 Fuel rod diagram for assembly 710

MONTFRINGS PROTOKOLL

PATRÖN NR. 710.

DATUM: 12-02-73

REF. HÖR!

SPRIDARHÄLLARSTAV 780911

```
*****
* 1.85 1.85 1.85 1.85 1.85 1.40 1.40 1.40 *
* 2.941 2.955 3.251 3.259 3.255 3.236 2.951 2.943 *
* 9117 9217 27485 27421 27420 21438 7116 7032 *
*
* 2.50 2.50 2.50 2.50 1.85 1.85 1.85 1.40 *
* 2.941 3.253 3.247 3.255 3.264 3.241 3.267 2.941 *
* 11055 31649 31629 31429 27474 479 27483 7163 *
*
* 2.50 2.50 2.50 2.50 2.50 2.50 1.85 1.40 *
* 3.257 3.242 3.259 3.245 3.247 3.250 3.253 3.255 *
* 31641 1610 31634 31492 31643 31518 27501 21498 *
*
* 2.50 2.50 2.50 2.50 2.50 2.50 1.85 1.85 *
* 3.259 3.253 3.250 3.256 3.246 3.257 3.258 3.261 *
* 31603 31556 31540 31642 31648 31646 27430 27504 *
*
* 2.50 2.50 2.50 2.50 2.50 2.50 2.50 1.85 *
* 3.256 3.250 3.260 3.253 3.251 3.255 3.255 3.255 *
* 31572 31609 31537 31615 31588 31613 27449 *
*
* 2.50 2.50 2.50 2.50 2.50 2.50 2.50 1.85 *
* 3.255 3.248 3.251 3.262 3.244 3.260 3.257 3.256 *
* 31536 31597 31636 31520 31539 31577 1693 27484 *
*
* 2.50 2.51 2.50 2.50 2.50 2.50 2.50 1.85 *
* 2.948 3.261 3.252 3.251 3.261 3.250 3.254 2.942 *
* 11074 31631 1600 31529 31527 31517 31535 9170 *
*
* 1.85 2.50 2.50 2.50 2.50 2.50 2.50 1.85 *
* 2.945 2.933 3.247 3.249 3.255 3.260 2.943 2.946 *
* 9268 11044 31426 31530 31614 31617 11073 9241 *
*****
```

ANRIK.N.	ANTAL	KG	KG	KG
NOM VERKE. STAVAR		U-2	U	U-235
1.40	1.412	5	15.326	13.502
1.85	1.853	16	50.549	44.534
2.50	2.504	42	135.383	119.272
S:A	2.257	63	201.258	177.308
				4.003

SPRIDARE NR.: 3180 3258 3200 3186 3265 3182 URSPR. LAND FÖR URAH: US
BOXNR.: 662

Ringhals 1 Fuel rod diagram for assembly 900

ROD TYPER I O S P R O T O K O L L

PATRON NR, 900, DATUM:10-03-73 REF.4680

```
*****
* 1.85  1.40  1.85  1.85  1.85  1.40  1.40  1.40 *
* 2.945 2.942 3.254 3.257 3.255 3.252 2.948 2.957 *
* 3759  3777  18297 16031 17150 22555 7509 7590 *
*
* 2.50  2.50  2.50  2.50  1.85  1.85  1.85  1.40 *
* 2.937 3.252 3.255 3.258 3.256 3.259 3.257 2.960 *
* 12076 40185 40183 40190 16857 343 15222 7574 *
*
* 2.50  2.50  2.50  2.50  2.50  2.50  1.85  1.40 *
* 3.263 3.240 3.262 3.246 3.259 3.260 3.261 3.255 *
* 40118 1205 40120 40116 40103 40147 17253 22511 *
*
* 2.50  2.50  2.50  2.50  2.50  2.50  1.85  1.85 *
* 3.263 3.251 3.257 3.254 3.240 3.250 3.265 3.248 *
* 46117 46177 40171 40167 40181 40168 16335 15567 *
*
* 2.50  2.50  2.50  2.50  2.50  2.50  2.50  1.85 *
* 3.247 3.255 3.254 3.256 3.249 3.255 3.251 3.265 *
* 39983 40006 39984 39981 40108 40189 40180 15071 *
*
* 2.50  2.50  2.50  2.50  2.50  2.50  2.50  1.85 *
* 3.267 3.266 3.255 3.262 3.259 3.252 3.246 3.249 *
* 39999 39976 39995 39998 40003 40187 1266 17002 *
*
* 2.50  2.50  2.50  2.50  2.50  2.50  2.50  1.85 *
* 2.944 3.249 3.246 3.242 3.255 3.256 3.255 2.942 *
* 12059 39997 1207 39994 40026 39996 39987 3716 *
*
* 1.85  2.50  2.50  2.50  2.50  2.50  2.50  1.85 *
* 2.950 2.939 3.253 3.245 3.254 3.251 2.943 2.936 *
* 8654 12026 40090 40178 40191 40010 12035 8697 *
*****
ANRIKH.3  ANTAL  KG  KG  KG
NOJ VERKL. STAVAR  U-02  U  U-235
1.40 1.409 5 15.582 13.552 0.191 TOPPL.NR.:40661
1.85 1.858 16 50.512 44.501 0.827
2.50 2.503 43 138.561 122.178 3.059 BOTTL.NR.:40690

STA 2.262 64 204.575 180.231 4.077
```

SPRIDARE NR.: 1522 1730 1653 1050 1023 1024 ONSPR.LAND FOR DRAG: JS
BOX NR.: 971

Ringhals 1 Fuel rod diagram for assembly 900

HÖUTERINGS PROTOKOLL

PATRON NR, 900, DATUM: 15-03-73 REF. 408N
 SPRIDARHALLARSTAV 78-08-22

* 1.85	1.85	1.85	1.85	1.85	1.40	1.40	1.40	*
* 2.945	2.942	3.254	3.257	3.255	3.252	2.948	2.957	*
* 8739	8777	16297	16331	17150	22555	7569	7590	*
* 2.50	2.50	2.50	2.50	1.85	1.85	1.85	1.40	*
* 2.937	3.252	3.255	3.258	3.256	3.250	3.257	2.960	*
* 12076	40185	40180	40190	16857	343	15222	7574	*
* 2.50	2.50	2.50	2.50	2.50	2.50	1.85	1.40	*
* 3.263	3.240	3.262	3.246	3.259	3.260	3.261	3.255	*
* 40118	1205	40120	40116	40103	40147	17253	22511	*
* 2.50	2.50	2.50	2.50	2.50	2.50	1.85	1.85	*
* 3.263	3.251	3.257	3.254	3.248	3.259	3.265	3.248	*
* 40117	40177	40171	40167	40181	40168	16335	15587	*
* 2.50	2.50	2.50	2.50	2.50	2.50	2.50	1.85	*
* 3.247	3.255	3.254	3.249	3.255	3.251	3.251	3.255	*
* 39983	40006	39984	40108	40189	40156	40156	15071	*
* 2.50	2.50	2.50	2.50	2.50	2.50	2.50	1.85	*
* 3.267	3.266	3.258	3.262	3.259	3.252	3.246	3.249	*
* 39999	39976	39995	39998	40008	40187	1266	17002	*
* 2.50	2.50	2.50	2.50	2.50	2.50	2.50	1.85	*
* 2.944	3.242	3.246	3.242	3.255	3.256	3.255	2.942	*
* 12059	39997	1207	39994	40026	39996	39987	5716	*
* 1.85	2.50	2.50	2.50	2.50	2.50	2.50	1.85	*
* 2.950	2.939	3.253	3.248	3.254	3.251	2.948	2.936	*
* 8654	12026	40090	40178	40191	40018	12035	8697	*

NOJ VERK, STAVAR	ANTAL	KG	KG	KG	TOPPL.NR.:	BOTTPL.NR.:
1.40	1.499	5	15.382	13.552	0.191	40663
1.85	1.858	16	50.512	44.501	0.827	
2.50	2.503	42	135.425	119.309	2.986	40690
SIA	2.258	63	201.319	177.362	4.004	

SPRIDARE NR.: 1592 1738 1653 1056 1023 1024 UNSPR.LAND FÖR URAN: JS
 BOX NR.: 971

Ringhals 1 Fuel rod diagram for assembly 1136

HÖRTERINGSPROTOKOLL

PATRON NR. 1136.

DATUM: 63-06-73

REF. 43RM

* 1.85	1.85	1.85	1.35	1.85	1.40	1.40	1.40	*
* 2.951	2.941	3.260	3.202	3.247	3.251	2.940	2.950	*
* 9373	9319	45913	45902	45877	10247	7721	7717	*
* 2.50	2.50	2.50	2.50	1.85	1.85	1.85	1.40	*
* 2.945	3.254	3.251	3.236	3.260	3.262	3.242	2.954	*
* 11071	29233	29360	29304	45910	194	45852	7741	*
* 2.50	2.50	2.50	2.50	2.50	2.50	1.85	1.40	*
* 3.250	3.241	3.251	3.236	3.239	3.244	3.261	3.266	*
* 29280	866	29220	29354	29326	29178	45923	10211	*
* 2.50	2.50	2.50	2.50	2.50	2.50	1.85	1.35	*
* 3.243	3.250	3.253	3.245	3.247	3.256	3.251	3.262	*
* 29219	29301	29225	29217	29401	29330	45913	45979	*
* 2.50	2.50	2.50	2.50	2.50	2.50	2.50	1.85	*
* 3.240	3.248	3.245	3.239	3.246	3.241	3.253	3.257	*
* 29250	29230	29270	29369	29365	29185	29337	45925	*
* 2.50	2.50	2.50	2.50	2.50	2.50	2.50	1.35	*
* 3.243	3.244	3.243	3.232	3.259	3.250	3.250	3.254	*
* 29389	29277	29353	29243	29303	29223	872	45965	*
* 2.50	2.50	2.50	2.50	2.50	2.50	2.50	1.85	*
* 2.951	3.240	3.256	3.241	3.245	3.242	3.237	2.950	*
* 11956	29248	869	29406	29307	29267	29419	9340	*
* 1.85	2.50	2.50	2.50	2.50	2.50	2.50	1.85	*
* 2.951	2.939	3.243	3.253	3.250	3.260	2.951	2.944	*
* 9400	11963	29370	29284	29293	29346	11973	9436	*

ÅHRIKN. #	ANTAL	KG	KG	KG	
HÖR VERKL. STAVAR		UO2	H	H-235	
1.40	1.407	5	15.361	13.533	0.190
1.85	1.853	16	50.563	44.546	0.825
2.50	2.509	43	130.407	121.937	3.059
SIA	2.263	64	204.331	180.016	4.074

SPRITADE NR.: 231 137 107 202 211 3230 MUSPELÄND F&O BRACK USA
 004NR.: 856

Ringhals 1 Fuel rod diagram for assembly 1136

MONTERINGS PROTOKOLL		RINGHALS 1					
PATRON NR	1136	810403					
1.85	1.85	1.85	1.85	1.85	1.40	1.40	1.40
2.951	2.941	3.260	3.262	3.247	3.251	2.940	2.950
9373	9319	45913	45902	45877	18247	7721	7717
2.50	2.50	2.50	2.50	1.85	1.85	1.85	1.40
2.945	3.254	3.251	3.236	3.260	3.262	3.242	2.954
11871	29233	29368	29304	45910	194	45852	7740
2.50	2.50	2.50	2.50	2.50	2.50	1.85	1.40
3.250	3.241	3.251	3.236	3.239	3.244	3.261	3.266
29280	866	29220	29364	29326	29178	45923	18211
2.50	2.50	2.50	2.50	2.50	2.50	1.85	1.85
3.243	3.258	3.253	3.246	3.247	3.256	3.251	3.262
29219	29301	29225	29217	29401	29330	45918	45970
2.50	2.50	2.50	.00	2.50	2.50	2.50	1.85
3.248	3.248	3.245	.000	3.246	3.241	3.253	3.265
29250	29230	29278		29365	29185	29337	45925
2.50	2.50	2.50	2.50	2.50	2.50	2.50	1.85
3.243	3.244	3.243	3.232	3.259	3.250	3.259	3.254
29389	29277	29353	29246	29300	29223	872	45965
2.50	2.50	2.50	2.50	2.50	2.50	2.50	1.85
2.951	3.240	3.256	3.241	3.245	3.242	3.237	2.950
11956	29248	869	29406	29307	29267	29410	9348
1.85	2.50	2.50	2.50	2.50	2.50	2.50	1.85
2.951	2.939	3.243	3.253	3.250	3.260	2.950	2.944
9409	11963	29379	29284	29293	29346	11973	9436
U02(KG)	U(KG)	U235(KG)					
201.092	177.161	4.008					

Ringhals 1 Fuel rod diagram for assembly 1136

MONTERINGSPROTOKOLL <i>Ringhals 1</i>							
PATRON NR	1136	810914	<i>CA</i>				
2.50	2.50	2.50	2.50	2.50	2.50	2.50	2.50
3.250	3.250	3.256	3.257	3.254	3.250	3.247	2.945
29280	44710	44721	44701	44702	44722	44781	11871
2.50	2.50	2.50	2.50	2.50	1.85	2.50	2.50
3.255	3.243	3.251	3.261	3.243	3.262	3.254	3.248
44707	29219	29368	44706	29389	194	44769	44727
2.50	2.50	2.50	2.50	2.50	2.50	1.85	2.50
3.256	3.241	3.251	3.236	3.239	3.258	3.261	3.261
44763	866	29220	29364	29326	44703	45923	44724
2.50	2.50	2.50	2.50	2.50	2.50	2.50	2.50
3.249	3.258	3.253	3.246	3.247	3.256	3.243	3.255
44736	29301	29225	29217	29401	29330	29379	44709
2.50	2.50	.00	.00	2.50	2.50	2.50	2.50
3.248	3.248	.000	.000	3.246	3.241	3.260	3.263
29250	29230			29365	29185	44776	44784
2.50	2.50	.00	.00	2.50	2.50	2.50	2.50
3.253	3.244	.000	.000	3.259	3.250	3.259	3.246
44718	29277			29300	29223	872	44726
2.50	2.50	2.50	2.50	2.50	2.50	2.50	2.50
3.245	3.258	3.256	3.241	3.245	3.242	3.250	3.251
29278	44723	869	29406	29307	29267	29293	44730
2.50	2.50	2.50	2.50	2.50	2.50	2.50	2.50
3.243	3.252	3.262	3.253	3.259	3.260	3.248	3.260
29353	29246	44788	29284	44786	44791	44740	29346
UO2(KG)	U(KG)	U235(KG)					
194.758	171.581	4.274					

Ringhals 1 Fuel rod diagram for assembly 1136

MONTERINGS PROTOKOLL

PATRON NR. 1136.

DATE: 03-06-73
SPRIDARHALLARSTAV 780906

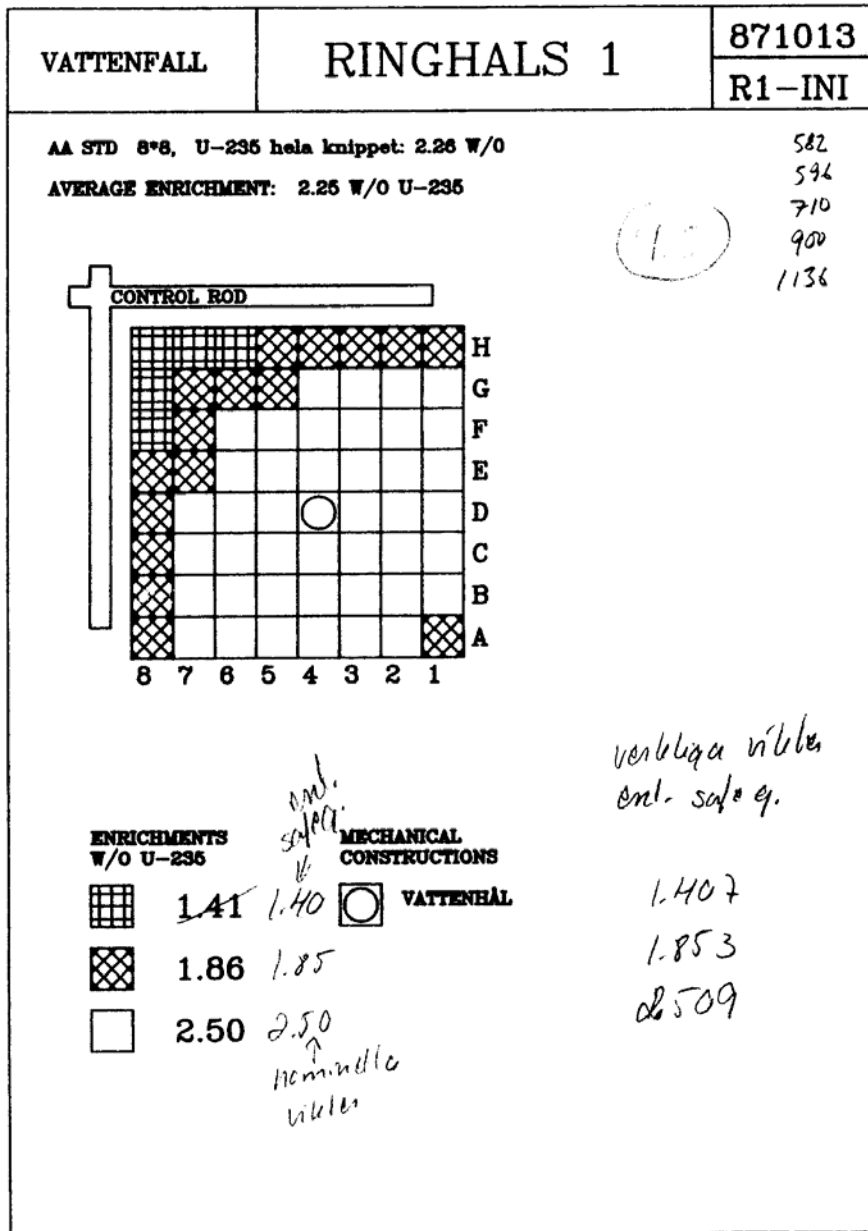
REF. 13RN

* 1.85	1.85	1.85	1.35	1.95	1.40	1.40	1.40	*
* 2.951	2.941	3.260	3.202	3.247	3.251	2.940	2.950	*
* 9373	9319	45913	45902	45877	18247	7721	7717	*
* 2.50	2.50	2.50	2.50	1.85	1.85	1.85	1.40	*
* 2.945	3.254	3.251	3.236	3.260	3.262	3.242	2.954	*
* 11671	29233	29360	29304	45910	194	45852	7740	*
* 2.50	2.50	2.50	2.50	2.50	2.50	1.85	1.40	*
* 3.250	3.241	3.251	3.236	3.239	3.244	3.261	3.266	*
* 29200	866	29220	29364	29326	29178	45923	18211	*
* 2.50	2.50	2.50	2.50	2.50	2.50	1.85	1.85	*
* 3.243	3.258	3.253	3.246	3.247	3.256	3.251	3.262	*
* 29219	29301	29225	29217	29401	29330	45918	45970	*
* 2.50	2.50	2.50	2.50	2.50	2.50	2.50	1.85	*
* 3.248	3.246	3.245	3.246	3.246	3.241	3.253	3.255	*
* 29250	29230	29278	29365	29185	29337	45925		*
* 2.50	2.50	2.50	2.50	2.50	2.50	2.50	1.85	*
* 3.243	3.244	3.243	3.232	3.259	3.250	3.259	3.254	*
* 29389	29277	29353	29245	29301	29223	872	45955	*
* 2.50	2.50	2.50	2.50	2.50	2.50	2.50	1.85	*
* 2.951	3.240	3.256	3.241	3.245	3.242	3.237	2.950	*
* 11956	29248	869	29406	29307	29267	29410	9348	*
* 1.85	2.50	2.50	2.50	2.50	2.50	2.50	1.85	*
* 2.951	2.939	3.243	3.253	3.250	3.260	2.950	2.944	*
* 9409	11963	29370	29284	29293	29346	11973	9436	*

ANRIKIL-%	ANTAL	KG	KG	KG	
NOH VERKL.	STAVAR	U02	U	U-235	
1.40	1.407	5	15.361	13.533	0.190
1.85	1.853	16	50.563	44.546	0.825
2.50	2.509	42	135.168	119.083	2.988
SIA	2.259	63	201.092	177.162	4.003

SPRIDARE NR. 1 237 137 107 207 231 3930 HESPR. LAND FOR DRANS JS.
BOXER. 1 856

Ringhals 1 Enrichment diagram for assemblies 582, 596, 710, 900 och 1177



Ringhals 1 Fuel rod diagram for assembly 1177

R O D D I A G R A M F O R A S S E M B L Y

RÖD 1177

BYGGNING 1177

DATUM 79-07-01

P. 077, 600

* 1.40	2.05	2.05	2.05	2.05	2.05	1.40	1.40	*
* 3.003	3.003	3.003	3.003	3.003	3.003	3.003	3.003	*
8* 5294	5473	15654	15736	15432	13703	5228	5228	*
*								*
*								*
* 2.05	2.05	2.02	3.10	3.10	2.82	2.05	1.40	*
* 3.004	3.329	3.304	3.313	3.319	3.300	3.322	3.014	*
7* 5479	15299	13700	16742	16537	4	15410	5293	*
*								*
*								*
* 2.82	3.10	3.10	3.10	3.10	3.100A	2.82	2.05	*
* 3.314	3.303	3.330	3.312	3.308	3.293	3.310	3.331	*
6* 15713	93	16307	16642	16613	19043	15699	15439	*
*								*
*								*
* 3.10	3.100A	3.10	3.10	3.10	3.10	3.10	2.05	*
* 3.314	3.292	3.313	3.329	3.314	3.311	3.317	3.323	*
5* 16696	19046	16456	16261	16773	16728	16641	15419	*
*								*
*								*
* 3.10	3.10	3.10		3.10	3.10	3.10	2.05	*
* 3.328	3.311	3.314		3.317	3.307	3.325	3.323	*
4* 16794	16721	16653	7074	16706	16640	16349	15440	*
*								*
*								*
* 2.82	3.10	3.10	3.10	3.10	3.10	2.82	2.05	*
* 3.311	3.316	3.308	3.315	3.304	3.310	3.309	3.320	*
3* 15788	16176	16577	16719	16620	16640	176	15421	*
*								*
*								*
* 2.05	2.82	3.10	3.10	3.100A	3.10	2.05	2.05	*
* 3.005	3.315	3.312	3.313	3.283	3.311	3.321	3.014	*
2* 5519	15680	101	16713	19047	16712	15404	5460	*
*								*
*								*
* 2.05	2.05	2.82	3.10	3.10	2.82	2.05	1.40	*
* 3.009	3.010	3.312	3.312	3.302	3.324	3.005	3.016	*
1* 5528	5498	15781	16720	16659	15733	5467	5254	*

A	B	C	D	E	F	G	H	
ANRIEN. Y	ANTAL	KG	KG	KG				
NOH	VERKL	STAVAN	002	0	0-235			
1.40	1.400	5	15.039	13.267	0.147			
2.05	2.000	18	57.418	50.741	1.039			
2.82	2.810	9	29.137	26.260	0.709			
3.10	3.000	24	92.721	81.349	3.081			
3.1000	3.100	3	9.573	0.534	0.208			
SUM	2.340	60	203.888	182.651	6.184			
TOTALMANTAL: 61300								
SÖTYPLENTAL: 53781								

SPRIBARS PRIS 16-0053 16-0079 16-1147 16-2003 16-0076 16-0060

FOR NÄR: 250784

OMSPÅLARB FOR URINER USA

16-0046 16-008 002 008200

Ringhals 1 Fuel rod diagram for assembly 1186

M O N T E R I N G S P R O T O K O L L

PATRON NR 1186

RINGHALS 1 ERS. 1 BA

DATUM: 76-05-07

REF. HÖRN

```

*****
* 1.40 2.05 2.05 2.05 2.05 2.35 1.40 1.40 *
* 3.012 3.006 3.325 3.322 3.307 3.329 3.014 3.010 *
8* 5393 5115 15229 15230 15268 15328 5298 5305 *
* * * * * * * * *
* 2.05 2.05 2.82 3.10 3.10 2.82 2.05 1.40 *
* 3.004 3.308 3.302 3.328 3.323 3.306 3.314 3.002 *
7* 5096 15329 15826 16303 16786 40 15377 5242 *
* * * * * * * * *
* 2.82 3.10 3.10 3.10 3.10 3.10BA 2.82 2.05 *
* 3.311 3.306 3.312 3.327 3.312 3.282 3.308 3.312 *
6* 15678 139 16678 15995 16658 19033 15700 15293 *
* * * * * * * * *
* 3.10 3.10BA 3.10 3.10 3.10 3.10 3.10 2.05 *
* 3.312 3.290 3.313 3.304 3.318 3.323 3.303 3.331 *
5* 16690 19132 16800 16747 16133 16175 16673 15431 *
* * * * * * * * *
* 3.10 3.10 3.10 3.10 3.10 3.10 3.10 2.05 *
* 3.313 3.323 3.314 7065 3.308 3.318 3.317 3.323 *
4* 16756 16325 16186 16592 16135 16688 15346 *
* * * * * * * * *
* 2.82 3.10 3.10 3.10 3.10 3.10 2.82 2.05 *
* 3.310 3.325 3.318 3.314 3.320 3.307 3.311 3.319 *
3* 15834 16208 16167 16629 16631 16657 57 15348 *
* * * * * * * * *
* 2.05 2.82 3.10 3.10 3.10BA 3.10 2.05 2.05 *
* 3.005 3.312 3.302 3.311 3.288 3.315 3.320 3.002 *
2* 5086 15716 190 16674 19024 16342 15361 5112 *
* * * * * * * * *
* 2.05 2.05 2.82 3.10 3.10 2.82 2.05 1.40 *
* 3.019 3.003 3.323 3.308 3.307 3.312 3.009 3.008 *
1* 5081 5045 15868 16760 16598 15734 5031 5418 *
*****

```

A		B		C		D		E		F		G		H	
ANRI.N.%	ANTAL	KG	KG	KG	KG	KG	KG	KG	KG	KG	KG	KG	KG	KG	KG
NOM	VEPKL	STAVAR	UO2	U	U-235										
1.40	1.409	5	15.046	13.256	0.187										
2.05	2.086	18	57.558	50.709	1.058										
2.82	2.813	9	29.795	26.249	0.738										
3.10	3.079	28	92.801	81.758	2.517										
3.10BA	3.100	3	9.860	8.543	0.265										
S:A	2.640	63	205.060	180.515	4.765										

TOPPL.NR.: 6036B
BOTTPL.NR.: 4586D

SPRIDARE NR.: 14-005 14-008 14-028 14-110 14-009 14-414
 BOX NR.: 2653
 URSPR.LAND FÖR URAN: USA
 INBLANDAD MÄNGD G02 03=2.0%

Ringhals 1 Enrichment diagram for assembly 1177 and 1186

1177

1186

VATTENFALL

RINGHALS 1

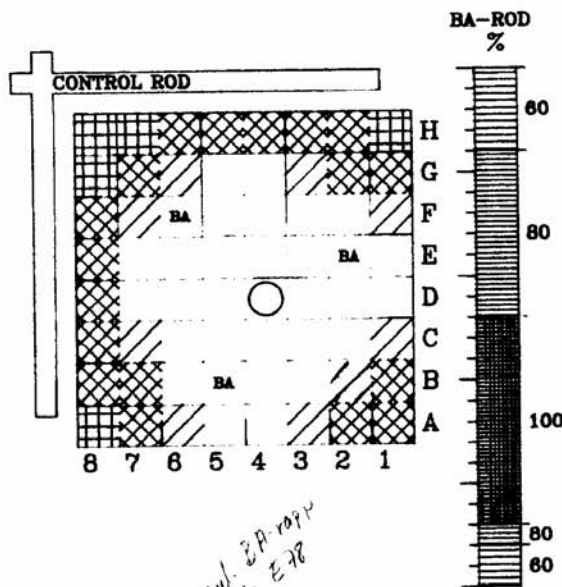
870923

R1-1B2

AA STD 8*8, 4*2.00 W/O BA, U-235 hela knippet: 2.64 W/O

AVERAGE ENRICHMENT: 2.63 W/O U-235

BA-LAYOUT: 3 * 2.00 W/O



1.6

ENRICHMENTS
W/O U-235



1.41

1.40



2.09

2.05



2.81

2.82



3.08

3.0

cul. sa/a9.

MECHANICAL
CONSTRUCTIONS



VATTENHÅL

prickstav

BA W/O AND TYPE



2.00 W/O
GD203

*cul. 2A-ropp
for E78*

*nominella vilken
cul. sa/a9.*

Ringhals 1 Fuel rod diagram for assembly 5829

PBB2		Riers 2 Exp-patr.		5829		sid 1		
Poppl.mont.sign.datum 79 06 07	Toppl.mont.godk. 790615	Mont.Sign.Dat. 790618	PATRON Kontr.Sign.Dat. Dokum.Sign.Dat. 790618					
Benämning	Materialkod	Toppl.nr M 203	XX Anrikning					
Mid-Grip M5 ritn.nr AA 131.650-2	81 BM 8541 pos 2:3	Spridare nr 130 6	XXXX Stavnr.					
Skruv M5 ritn.nr AA 131.691	81 BM 8485:5/895	195 5	Bärande stav					
Bladjäder tunn ritn.nr AA 131.730	81 BM 8381 pos 3:2/255	211 4	Spridarhållarstav					
Bladjäder tjock ritn.nr AA 131.721	81 BM 8381 pos 2:2/239	225 3						
Bladjäder tjock ritn.nr AA 147.718	81 BM 8381 pos 4:2/218	187 2	Stav med mindre diameter Pilens riktning på topplattan=referens- hörn					
Lösfjäder ritn.nr AA 147.782	92 RF 3534 pos 1/255	226 1						
Mutter M8 ritn.nr AA 101.538	M8-8	Bottenpl.nr Z 2103	Stav med mindre diameter Pilens riktning på topplattan=referens- hörn					
Bladjäder yttre ritn.nr AA 147.760	81 BM 8551:1/232	71 B 080						
Mutter Topplatta ritn.nr AA 172.528	92 RF 3534 pos 2	Box nr. 71 B 080	Stav med mindre diameter Pilens riktning på topplattan=referens- hörn					
Ver M6 ritn.nr AA 147.753	M6-2							
8	1.82 2940 6680	2.34 2941 6673	2.34 3258 25534	2.34 3263 25494	2.34 3254 25883	1.82 3248 25857	1.82 2944 6670	1.82 2956 6687
7	2.34 2956 6659	3.17/1.28 2311/344 79T 8	3.17 3260 25906	3.17 2796 541	3.17 3263 25914	2.34 3258 25852	2.34 3263 25923	1.82 2936 6661
6	3.17/1.28 2087/344 79T 24	3.17Ba 3244 25911	3.17 3251 25853	3.17 3016 25550	3.17 2787 25555	3.17Ba 3242 25957	2.34 3254 25873	1.82 3251 25875
5	3.17/1.28 2300/344 79T 19	3.17 3253 546	3.17 3253 25890	1.82 3246 25888	1.82 3251 25874	3.17 2790 25563	3.17 3261 25934	2.34 3261 25881
4	3.17 3253 25920	3.17 3262 25889	3.17 3261 25905	3.17 39 063	1.82 3254 25951	3.17 2983 25559	3.17 2794 537	2.34 3256 25880
3	3.17/1.28 2309/344 79T 9	3.17 3007 25560	3.17 3259 25921	3.17 3263 25936	3.17 3252 25944	3.17 3253 25876	3.17 3260 25931	2.34 3261 25540
2	2.34 2944 6667	3.17Ba 3248 25912	3.17 2977 25558	3.17 3261 25558	3.17 3261 25558	3.17Ba 3243 25558	3.17/1.28 2302/344 25558	2.34 2944 6667
1	2.34 2952 6678	2.34 2948 6653	3.17/1.28 2306/347 79T 11	3.17 3261 25954	3.17/1.28 2311/344 79T 14	3.17/1.28 2159/346 79T 22	2.34 2952 6652	1.82 2948 6654

Ringhals 1 Fuel rod diagram for assembly 5829

sid 2

MONTERINGSKONTROLL RINGHALS 1

PRYÖR NR 3829 011731

1.82	2.34	2.34	2.34	2.34	1.82	1.82	1.38
2.940	2.941	3.258	3.258	3.258	3.248	2.944	2.950
30-6660	30-6673	30-25534	30-25494	30-25494	30-25883	30-6670	30-6667
2.34	3.17	3.17	3.17	3.17	2.34	2.34	1.82
2.956	2.971	3.260	2.796	3.263	3.258	3.263	2.956
30-6659	30-7475A	30-25906	30-541	30-25914	30-25852	30-25923	30-6661
3.17	3.17	3.17	3.17	3.17	3.17	2.34	1.82
2.067	3.244	3.251	3.016	2.787	3.242	3.254	3.251
3079124A	30-25911	30-25553	30-25550	30-25555	30-25557	30-25873	30-25675
3.17	3.17	3.17	3.17	3.17	3.17	3.17	1.82
2.300	3.253	3.253	3.246	3.251	2.790	3.261	2.34
3079124A	30-540	30-25490	30-25668	30-25874	30-25503	30-25934	30-25681
3.17	3.17	3.17	3.17	3.17	3.17	3.17	2.34
3.253	3.262	3.261	3.000	3.254	2.983	2.794	3.256
30-25920	30-25909	30-25905	30-25951	30-25559	30-537	30-25580	30-25540
3.17	3.17	3.17	3.17	3.17	3.17	3.17	2.34
2.309	3.007	3.259	3.263	3.252	3.253	3.260	3.261
30-7471A	30-25500	30-25921	30-25936	30-25944	30-25676	30-25931	30-25540
2.34	3.17	3.17	3.17	3.17	3.17	3.17	2.34
2.944	3.248	2.977	3.261	3.261	3.243	2.302	2.944
30-6367	30-25916	30-25536	30-25933	30-549	30-25902	3079110A	30-6663
2.34	2.34	2.34	3.17	3.17	3.17	3.17	1.82
2.952	2.798	2.506	3.261	2.311	2.169	2.952	2.948
30-6678	30-6653	3079111A	30-25934	3079114A	3079122A	30-6652	30-6664
0.238	0.347	0.347	0.346	0.344	0.346	0.346	0.346
30-7471B	30-7471B	30-7471B	30-7471B	30-7471B	30-7471B	30-7471B	30-7471B
0.238	0.347	0.347	0.346	0.344	0.346	0.346	0.346
0.238	0.347	0.347	0.346	0.344	0.346	0.346	0.346

Ringhals 1 Fuel rod diagram for assembly 5829

sid 3

POSTERIORROTORROLL RINGHALS 1

PATRON NR 5629 811014

1.82	2.34	2.34	2.34	2.34	1.82	1.82	1.38
2.940	2.941	3.256	3.263	3.254	5.248	4.944	2.956
30-6680	30-6673	30-25534	30-25474	30-25883	30-25657	30-6670	30-6687
2.34	5.17	5.17	5.17	3.17	2.34	2.34	1.82
2.956	2.511	3.260	2.796	3.243	5.258	3.263	2.936
30-6657	30-75154	30-25306	30-541	30-25914	30-25852	30-25923	30-6661
5.17	5.17	5.17	5.17	5.17	3.17	2.34	1.82
2.087	3.244	3.251	3.016	2.787	3.242	3.254	3.251
30791248	30-25711	30-25853	30-25550	30-25555	30-25957	30-25873	30-25875
2.340	3.253	3.253	3.240	.000	2.790	3.261	3.261
30791194	30-546	30-25890	30-25888		30-25563	30-25934	30-25881
5.17	5.17	5.17	5.17	5.17	5.17	5.17	2.34
3.253	3.262	3.261	.000	3.254	2.963	2.794	3.256
30-25920	30-25869	30-25705		30-25951	30-25559	30-537	30-25880
3.17	3.17	3.17	3.17	3.17	3.17	3.17	2.34
3.309	3.007	3.659	3.663	3.252	3.253	3.260	3.261
30-79132	30-25360	30-25921	30-25956	30-25944	30-25876	30-25931	30-25340
2.34	3.17	3.17	3.17	3.17	3.17	3.17	2.34
2.944	3.246	2.977	5.201	3.241	5.243	2.306	2.944
30-6367	30-25416	30-25356	30-25955	30-549	30-25962	30791104	30-6683
2.34	2.34	5.17	5.17	3.17	5.17	5.17	1.82
2.952	2.948	4.500	3.291	2.311	2.109	2.952	2.948
30-6678	30-6655	30751134	30-25954	30791144	30791224	30-6652	30-6664
004 (KG)	045 (KG)	023 (KG)					
187.055	107.555	4.527					

Ringhals 1 Fuel rod diagram for assembly 5829

sid4

UTSKRIFTSDATUM: 1987-09-30

Ringhals 1 KOLL 1 RINGHALLSPROVOKOLL FOR PATRON: 5829

DATUM: 870922

ORR: B FOS 1165

BOX-NR:

	A	B	C	D	E	F	G	H
6	1.82	2.34	2.34	2.34	2.34	1.82	1.82	1.38
	30-6680	30-6673	30-25536	30-25494	30-25533	30-6670	30-6687	
7	2.34	3.17	3.17	3.17	3.17	2.34	2.34	1.82
	30-6658	30-7918	30-25906	30-541	30-25914	30-25852	30-25923	30-6661
8	3.17	3.17	3.17	3.17	3.17	3.17	2.34	
	30-79124	30-25911	30-25853	30-25850	30-25955	30-25957	30-25873	
5	3.17	3.17	3.17	1.82	3.17	3.17	3.17	2.34
	30-546	30-25859	30-25856	30-25859	30-25859	30-25859	30-25934	30-25881
4	3.17	3.17	3.17	3.17	1.82	3.17	3.17	2.34
	30-25929	30-25859	30-25945	30-25945	30-25951	30-25559	30-537	30-25880
3	3.17	3.17	3.17	3.17	3.17	3.17	3.17	2.34
	30-7919	30-25540	30-25921	30-25936	30-25944	30-25876	30-25931	30-25540
2	2.34	3.17	3.17	3.17	3.17	3.17	3.17	2.34
	30-6357	30-25916	30-25858	30-25933	30-549	30-25962	30-79110	30-6683
3	2.34	3.17	3.17	3.17	3.17	3.17	3.17	1.82
	30-6676	30-6653	30-79111	30-25954	30-79114	30-79122	30-6652	30-6664

Ringhals 1 Fuel rod diagram for assembly 5829

sid 5

RINGHALS BLOCK 1 MONTEL: ISPROTO LL FOR PATRON: 5829 *SKRIFT ATUM: 1989-10-06
DATUM: 880113 OMR: BB POS: E165 BOX-NR: E

	A	B	C	D	E	F	G	H
8	* 1.82	* 2.34	* 3.17	* 3.17	* 3.17	* 2.34	* 1.82	* 1.38
	* 30-6680	* 30-6673	* 30-25534	* 30-25494	* 30-25883	* 30-6670	* 30-6687	*
7	* 2.34	* 3.17	* 3.17	* 3.17	* 3.17	* 2.34	* 2.34	* 1.82
	* 30-6659	* 30-79T8	* 30-25906	* 30-541	* 30-25914	* 30-25852	* 30-25923	* 30-6661
6	* 3.17	* 3.17	* 3.17	* 3.17	* 3.17	* 3.17	* 2.34	* 2.34
	* 30-79T24	* 30-25911	* 30-25853	* 30-25550	* 30-25555	* 30-25957	* 30-25873	*
5	* 3.17	* 3.17	* 3.17	* 1.82	* 3.17	* 3.17	* 3.17	* 2.34
	* 30-546	* 30-25890	* 30-25888	*	* 30-25563	* 30-25934	* 30-25881	*
4	* 3.17	* 3.17	* 3.17	*	* 1.82	* 3.17	* 3.17	* 2.34
	* 30-25920	* 30-25889	* 30-25905	*	* 30-25951	* 30-25559	* 30-537	* 30-25880
3	* 3.17	* 3.17	* 3.17	* 3.17	* 3.17	* 3.17	* 3.17	* 2.34
	* 30-79T9	* 30-25560	* 30-25921	* 30-25936	* 30-25944	* 30-25876	* 30-25931	* 30-25540
2	* 2.34	* 3.17	* 3.17	* 3.17	* 3.17	* 3.17	* 3.17	* 2.34
	* 30-6367	* 30-25918	* 30-25558	* 30-25933	* 30-549	* 30-25962	* 30-79T10	* 30-6683
1	* 2.34	* 3.17	* 3.17	* 3.17	* 3.17	* 3.17	* 2.34	* 1.82
	* 30-6678	* 30-6653	* 30-79T11	* 30-25954	* 30-79T14	* 30-79T22	* 30-6652	* 30-6664

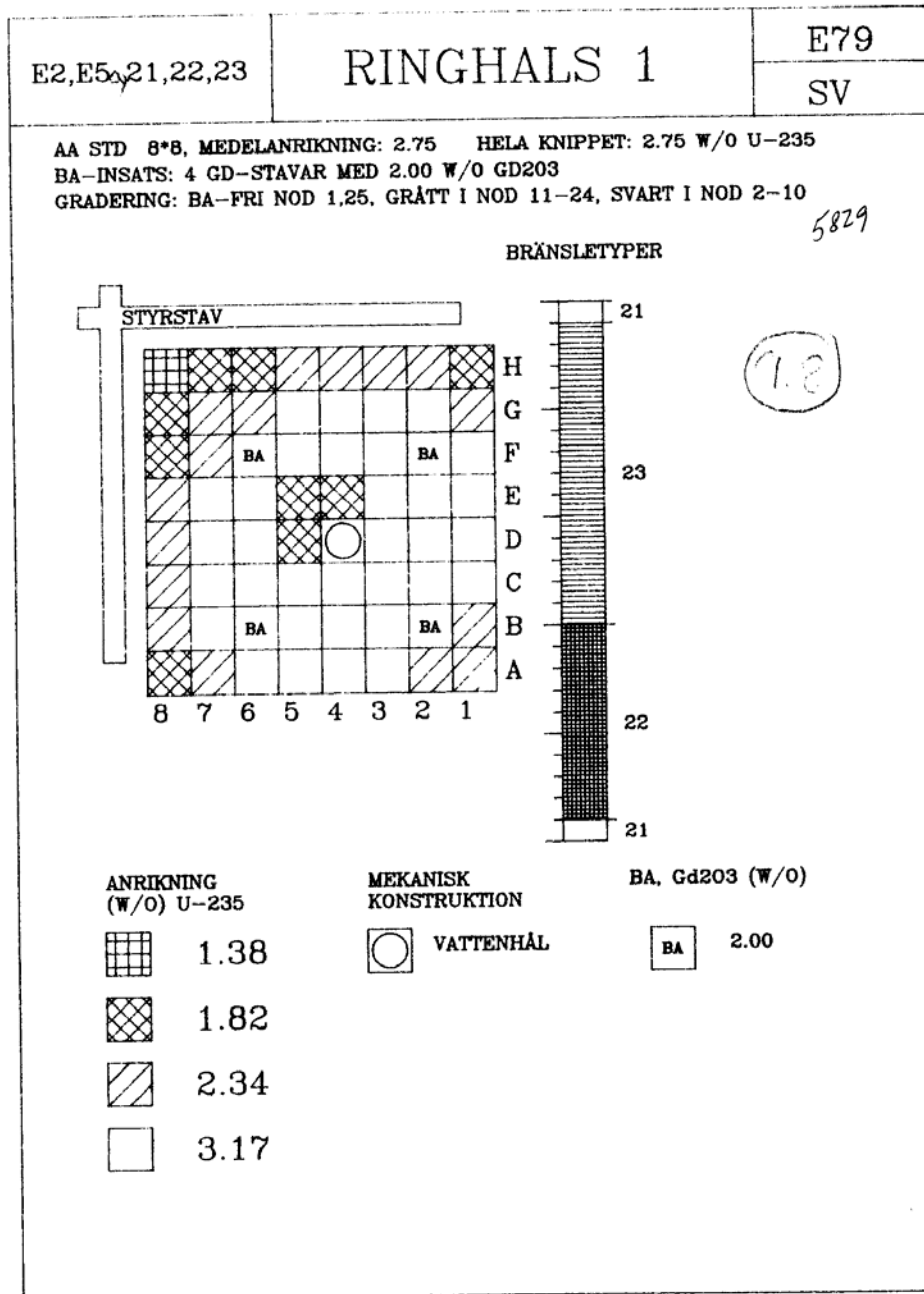
Ringhals 1 Fuel rod diagram for assembly 5829

sid 6

547)

RINGHALS BLOCK 1		MONTER SSPROTJ ALL FOR PATRON: 5829		TSKRIFT DATUM: 1989-10-12			
DATUM: 891011		OMR: 8p POS: E.65		BOX-NR:			
A	B	C	D	E	F	G	H
*	*	*	*	*	*	*	*
1.82	2.34	3.17	2.34	2.34	2.34	1.82	1.38
*	30-6680	*	30-25534	*	30-25494	*	30-6687
*	*	*	*	*	6	*	*
*	*	*	*	*	*	*	*
2.34	3.17	3.17	3.17	3.17	2.34	2.34	1.82
*	30-6659	*	30-79T8	*	30-25906	*	30-6661
*	*	*	*	*	30-541	*	30-25923
*	*	*	*	*	*	*	*
3.17	3.17	3.17	3.17	3.17	3.17	2.34	2.34
*	30-79T24	*	30-25911	*	30-25550	*	30-25873
*	*	*	*	*	30-25555	*	*
*	*	*	*	*	*	*	*
3.17	3.17	3.17	1.82	3.17	3.17	3.17	2.34
*	30-546	*	30-25890	*	30-25888	*	30-25881
*	*	*	*	*	*	*	*
3.17	3.17	3.17	3.17	1.82	3.17	3.17	2.34
*	30-25920	*	30-25889	*	30-25905	*	30-25880
*	*	*	*	*	30-25951	*	30-25880
*	*	*	*	*	*	*	*
3.17	3.17	3.17	3.17	3.17	3.17	3.17	2.34
*	30-79T9	*	30-25560	*	30-25921	*	30-25540
*	*	*	*	*	30-25936	*	30-25931
*	*	*	*	*	*	*	*
2.34	3.17	3.17	3.17	3.17	3.17	3.17	2.34
*	30-6367	*	30-25918	*	30-25558	*	30-6683
*	*	*	*	*	30-549	*	30-25962
*	*	*	*	*	*	*	*
2.34	2.34	3.17	3.17	3.17	3.17	3.17	1.82
*	30-6678	*	30-6653	*	30-79T11	*	30-6664
*	*	*	*	*	30-25954	*	30-6652
*	*	*	*	*	30-79T14	*	*
*	*	*	*	*	*	*	*

Ringhals 1 Enrichment diagram for assembly 5829



Ringhals 1 Fuel rod diagram for assembly 6423

10 N T E R I N G S P R O T O K O L L PATRON NR 6423

033 R1E3

REF. HÖRN

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*****
* 1.98 2.49 2.49 2.49 2.49 1.98 1.98 1.38 *
* 2.963 2.949 3.256 3.257 3.253 3.250 2.963 2.958 *
8* 5203 5116 21570 21549 21538 20559 5191 5488 *
*
* 2.49 3.37 3.17 3.37 3.37 2.49 2.49 1.98 *
* 2.954 3.264 3.250 3.264 3.263 3.252 3.254 2.959 *
7* 5132 21276 20494 73 21238 21571 21514 5199 *
*
* 3.37 3.17BA 3.37 3.37 3.37 3.17BA 2.49 1.98 *
* 3.268 3.262 3.261 3.265 3.263 3.263 3.261 3.263 *
6* 21305 15102 21215 21197 21192 15113 21560 20549 *
*
* 3.37 3.37 3.37 1.98 1.98 3.37 3.37 2.49 *
* 3.262 3.266 3.263 3.270 3.264 3.264 3.266 3.250 *
5* 21329 30 21287 20545 20599 21156 21217 21545 *
*
* 3.37 3.37 3.37 1.98 3.37 3.37 3.37 2.49 *
* 3.270 3.260 3.261 3.265 3.260 3.271 3.258 *
4* 21182 21298 21308 53117 20551 21310 60 21559 *
*
* 3.37 3.37 3.17 3.37 3.37 3.37 3.17 2.49 *
* 3.267 3.270 3.258 3.260 3.262 3.269 3.249 3.251 *
3* 21267 21285 20508 21335 21333 21315 20483 21544 *
*
* 2.49 3.17BA 3.37 3.37 3.37 3.17BA 3.37 2.49 *
* 2.950 3.270 3.263 3.260 3.265 3.266 3.265 2.951 *
2* 5169 15060 21317 21350 100 15132 21343 5155 *
*
* 2.49 2.49 3.37 3.37 3.37 3.37 2.49 1.98 *
* 2.950 2.955 3.262 3.260 3.264 3.262 2.953 2.965 *
1* 5131 5147 21334 21316 21264 21332 5139 5215 *
*****

```

	A	B	C	D	E	F	G	H
ANRIKN.%								
NOM	1.38	1.407						
VERKL	1.984							
STAVAR		1						
KG			2.953					
U2			28.162					
U			49.954					
U-235			13.061					
TOPPL.NR			9.757					
BOTTPL.NR			97.920					
BOX NR								

S:A 2.900 63 201.812 177.701 5.154
INBLANDAD MANGD GD 2.00%
SPRIDARE NR.: 33776 34011 34019 34024 34204 34220

GODK. FÖR INFrysNING OCH LEVERANS DAT. 1980-12-22 SIGN: *HLA*

Ringhals 1 Fuel rod diagram for assembly 6432

M O N T E R I N G S P R O T O K O L L PATRON NR 6432

033 R1E3
REF. HORN

* 1.98	2.49	2.49	2.49	2.49	1.98	1.98	1.38	*
* 2.956	2.949	3.256	3.258	3.264	3.253	2.965	2.955	*
8* 6283	6087	23042	23064	23068	22848	6362	6269	*
* 2.49	3.37	3.17	3.37	3.37	2.49	2.49	1.98	*
* 2.957	3.263	3.251	3.269	3.255	3.264	3.269	2.963	*
7* 6177	22197	22676	249	22179	23100	23070	6290	*
* 3.37	3.17BA	3.37	3.37	3.37	3.17BA	2.49	1.98	*
* 3.254	3.249	3.255	3.257	3.258	3.249	3.261	3.250	*
6* 22050	16112	22148	22048	22165	16116	23093	22824	*
* 3.37	3.37	3.37	1.98	1.98	3.37	3.37	2.49	*
* 3.263	3.269	3.254	3.250	3.256	3.257	3.254	3.263	*
5* 22175	207	22101	22841	22815	22188	22120	22646	*
* 3.37	3.37	3.37		1.98	3.37	3.37	2.49	*
* 3.256	3.261	3.254		3.255	3.263	3.264	3.257	*
4* 22105	22084	22090	53094	22805	22160	241	22649	*
* 3.37	3.37	3.17	3.37	3.37	3.37	3.17	2.49	*
* 3.265	3.260	3.258	3.252	3.255	3.264	3.248	3.259	*
3* 22161	22088	22707	22072	22021	22046	22680	22675	*
* 2.49	3.17BA	3.37	3.37	3.37	3.17BA	3.37	2.49	*
* 2.951	3.252	3.253	3.256	3.262	3.252	3.256	2.952	*
2* 6080	16117	22083	22017	257	16115	22074	6015	*
* 2.49	2.49	3.37	3.37	3.37	3.37	2.49	1.98	*
* 2.953	2.953	3.262	3.254	3.252	3.262	2.959	2.960	*
1* 6121	6045	22113	22103	22019	22020	6093	6264	*

A	B	C	D	E	F	G	H
ANRIKN.%	ANTAL	KG	KG	KG			
NOM	VERKL	UO2	U	U-235	TOPPL.NR	: M630	
1.38	1.407	2.955	2.605	0.037	BOTTPL.NR	: M1075	
1.98	1.984	28.108	24.774	0.492	BOX NR	: 73B162	
2.49	2.503	50.025	44.092	1.104			
3.17BA	3.177	13.002	11.284	0.359			
3.17	3.170	9.757	8.600	0.273			
3.37	3.334	97.759	86.165	2.873			
S:A	2.893	63	201.606	177.520	5.138		
INBLANDAD MANGD GD	2.00%						
SPRIDARE NR.:	32766	32902	32930	32939	32940	34799	

GODK. FÖR INFrysNING OCH LEVERANS DAT. 1990-12-22 SIGN: *HL*

Ringhals 1 Fuel rod diagram for assembly 6454

MONTERINGSPROTOKOLL PATRON NR 6454

 033 R1E3
 REF. HÖRN

*	1.98	2.49	2.49	2.49	2.49	1.98	1.98	1.38	*
*	2.963	2.954	3.258	3.267	3.255	3.252	2.957	2.958	*
8*	6324	6185	23359	23376	23374	23247	6375	6237	*
*									*
*	2.49	3.37	3.17	3.37	3.37	2.49	2.49	1.98	*
*	2.961	3.261	3.250	3.263	3.263	3.258	3.260	2.951	*
7*	6030	23291	23346	323	23268	23362	23365	6359	*
*									*
*	3.37	3.17BA	3.37	3.37	3.37	3.17BA	2.49	1.98	*
*	3.257	3.267	3.260	3.262	3.266	3.268	3.260	3.256	*
6*	23269	16020	23305	23275	23292	16044	23356	23241	*
*									*
*	3.37	3.37	3.37	1.98	1.98	3.37	3.37	2.49	*
*	3.260	3.260	3.306	3.256	3.252	3.254	3.264	3.258	*
5*	23265	319	23331	23254	23242	23303	23284	23355	*
*									*
*	3.37	3.37	3.37		1.98	3.37	3.37	2.49	*
*	3.267	3.263	3.259		3.257	3.266	3.265	3.261	*
4*	23289	23309	23285	53067	23244	23290	322	23370	*
*									*
*	3.37	3.37	3.17	3.37	3.37	3.37	3.17	2.49	*
*	3.259	3.264	3.251	3.265	3.257	3.258	3.250	3.262	*
3*	23307	23259	23347	23280	23293	23297	23340	23363	*
*									*
*	2.49	3.17BA	3.37	3.37	3.37	3.17BA	3.37	2.49	*
*	2.949	3.269	3.261	3.259	3.264	3.266	3.267	2.961	*
2*	6215	16011	23270	23278	326	16059	23276	6216	*
*									*
*	2.49	2.49	3.37	3.37	3.37	3.37	2.49	1.98	*
*	2.946	2.946	3.259	3.263	3.265	3.264	2.950	2.962	*
1*	6138	6035	23277	23281	23271	23264	6126	6365	*

	A	B	C	D	E	F	G	H
ANRIKN.%								
NOM	VERKL	ANTAL	STAVAR	KG	KG	KG		
1.38	1.407		1	2.953	U	U-235	TOPPL.NR : M517	
1.98	1.984		9	28.106	24.773	0.492	BOTTPL.NR: 24197	
2.49	2.503		16	50.006	44.075	1.103	BOX NR : 73B008	
3.17BA3	3.177		4	13.070	11.343	0.360		
3.17	3.170		3	9.751	8.595	0.272		
3.37	3.343		30	97.901	86.290	2.885		

S:A 2.898 63 201.792 177.683 5.149
 INBLANDAD MANGD GD 2.00%
 SPRIDARE NR.: 33738 36158 36418 36429 36437 36442

GODK. FÖR INFrysNING OCH LEVERANS DAT. 1990-12-22 SIGV. *HL*

Ringhals 1 Fuel rod diagram for assembly 6478

sid 1

MONTERINGS PROTOKOLL

PATRON NR 6478

033

R1E3

REF.IGRN

* 1.98	2.49	2.49	2.49	2.49	1.98	1.98	1.38	*
* 2.953	2.963	3.267	3.255	3.262	3.256	2.953	2.954	*
8* 8028	8031	25308	25317	25374	25425	8030	8003	*
* 2.49	3.37	3.17	3.37	3.37	2.49	2.49	1.98	*
* 2.959	3.262	3.256	3.234	3.261	3.262	3.258	2.950	*
7* 8089	25093	25515	504	25001	25316	25354	8026	*
* 3.37	3.17BA	3.37	3.37	3.37	3.17BA	2.49	1.98	*
* 3.259	3.244	3.260	3.258	3.265	3.250	3.255	3.261	*
6* 25008	18016	25002	25016	25004	18027	25328	25430	*
* 3.37	3.37	3.37	1.98	1.98	3.37	3.37	2.49	*
* 3.261	3.237	3.262	3.255	3.260	3.259	3.261	3.263	*
5* 25087	539	25023	25453	25432	25031	25117	25312	*
* 3.37	3.37	3.37		1.98	3.37	3.37	2.49	*
* 3.260	3.265	3.264		3.264	3.262	3.235	3.265	*
4* 25044	25137	25015	53065	25458	25022	535	25356	*
* 3.37	3.37	3.17	3.37	3.37	3.37	3.17	2.49	*
* 3.262	3.264	3.257	3.262	3.264	3.260	3.268	3.261	*
3* 25017	25021	25532	25101	25102	25061	25523	25338	*
* 2.49	3.17BA	3.37	3.37	3.37	3.17BA	3.37	2.49	*
* 2.964	3.244	3.264	3.264	3.237	3.248	3.264	2.963	*
2* 8149	18037	25089	25092	531	18026	25069	8124	*
* 2.49	2.49	3.37	3.37	3.37	3.37	2.49	1.98	*
* 2.956	2.960	3.257	3.266	3.267	3.265	2.962	2.950	*
1* 8123	3152	25110	25109	25139	25076	8133	8042	*

A B C D E F G H

ANRIKN.%	ANTAL	KG	KG	KG	TOPPL.NR
NOM	VERKL	UJ2	U	U-235	
1.38	1.407	2.954	2.604	0.037	M565
1.98	1.980	28.102	24.769	0.490	Z4203
2.49	2.506	50.075	44.136	1.106	73B270
3.17BA	3.177	12.986	11.271	0.358	
3.17	3.172	9.781	8.621	0.273	
3.37	3.348	97.761	86.167	2.885	

S:A 2.900 63 201.659 177.568 5.149
 INBÅLADAD MÅNGD GD 2.00%
 SPRIDARE NR.: 33767 34805 34807 34831 34833 34847

1980-12-22

GODK. FÖR INFrysNING OCH LEVERANS DAT.....SIGN:.....

Ringhals 1 Fuel rod diagram for assembly 6478

EJ GILTIKT - HISTORIK

B OMR: BB PUS: CC38 B SOX-NR: 31B271 E H

Fore control

8	1.98	2.49	2.49	2.49	1.98	1.98	1.98	1.98	1.98	33-8030	33-8003
	33-8028	33-8081	33-25308	33-25317	33-25374	33-25425	33-8030	33-8030	33-8030	33-8003	
7	2.49	3.37	3.37	3.37	3.37	2.49	2.49	2.49	2.49	33-8026	
	33-8039	33-25093	33-25515	33-504	33-25001	33-25316	33-25354	33-8026	33-8026		
6	3.37	3.17	3.37	3.37	3.37	3.17	2.49	2.49	2.49	33-25430	
	33-25008	33-18016	33-25002	33-25016	33-25004	33-18027	33-25328	33-25430	33-25430		
5	3.37	3.37	3.37	1.98	1.98	3.37	3.37	3.37	3.37	33-25312	
	33-25087	33-539	33-25023	33-25453	33-25432	33-25031	33-25117	33-25312	33-25312		
4	3.37	3.37	3.37	1.98	1.98	3.37	3.37	3.37	3.37	33-25356	
	33-25044	33-25137	33-25015	33-25458	33-25022	33-535	33-25356	33-25356	33-25356		
3	3.37	3.37	3.37	3.37	3.37	3.37	3.17	3.17	3.17	33-25338	
	33-25017	33-25021	33-25532	33-25101	33-25102	33-25061	33-25523	33-25338	33-25338		
2	2.49	3.17	3.37	3.37	3.37	3.17	3.37	3.37	3.37	33-8124	
	33-8149	33-18037	33-25089	33-25092	33-531	33-18026	33-25069	33-8124	33-8124		
1	2.49	2.49	3.37	3.37	3.37	3.37	2.49	2.49	2.49	33-8042	
	33-8123	33-8152	33-25110	33-25109	33-25139	33-25076	33-8133	33-8042	33-8042		

sid 2

Ringhals 1 Fuel rod diagram for assembly 6478

UTSKRIFTSDATUM: 1986-09-02

RINGHALS BLOCK 1 MONTERINGSPROTOKOLL FÖR PATRON: 6478
 DATUM: 860826 OMR: B POS: 3 BOX-NR: 318271

	A	B	C	D	E	F	G	H
8	1.98	2.49	2.49	2.49	2.49	1.98	1.98	1.38
	33-8028	33-8081	33-25308	33-25317	33-25374	33-25425	33-8030	33-8003
7	2.49	3.37	3.17	3.37	3.37	2.49	2.49	1.98
	33-8039	33-25093	33-25515	33-504	33-25001	33-25316	33-25354	33-8026
6	3.37	3.17	3.37	3.37	3.37	3.17	2.49	1.98
	33-25008	33-18016	33-25002	33-25016	33-25004	33-18027	33-25328	33-25430
5	3.37	3.37	1.98	1.98	1.98	3.37	3.37	2.49
	33-25087	33-539	33-25425	33-25432	33-25031	33-25117	33-25312	
4	3.37				1.98		3.37	2.49
	33-25044				33-25438		33-535	33-25356
3	3.37	3.17	3.37	3.37	3.37	3.17	3.17	2.49
	33-25017	33-25532	33-25101	33-25102	33-25061	33-25523	33-25338	
2	2.49	3.17	3.37	3.37	3.37	3.17	3.37	2.49
	33-8149	33-18037	33-25089	33-531	33-18026	33-25069	33-8124	
1	2.49	2.49	3.37	3.37	3.37	2.49	2.49	1.98
	33-8123	33-8152	33-25110	33-25109	33-25139	33-25076	33-8133	33-8042

sid 3

Ringhals 1 Fuel rod diagram for assembly 6478

sid 4

RINGHALS BLOCK 1 MONTERINGSROTOKOLL FÖR PATRON: 6478 UTSKRIFTSDATUM: 1987-09-30

DATUM: 970929 UMR: POS: -62- BOX-NR: E

	A	B	C	D	E	F	G	H
8	1.98	2.49	3.37	3.37	3.37	1.98	1.98	
	33-8026	33-8051	33-25003	33-25515	33-25001	33-25425	33-8030	
7	2.49	3.37	3.17	3.37	3.37	2.49	2.49	1.98
	33-8089	33-25003	33-25515	33-504	33-25001	33-25316	33-25354	33-8026
6	3.37	3.37	3.37	3.37	3.37	3.17	2.49	1.98
	33-25037	33-539	33-25002	33-25016	33-25004	33-18027	33-25328	33-25430
5	3.37	3.37	1.98	1.98	1.98	3.37	3.37	3.37
	33-25037	33-539	33-25453	33-25453	33-25452	33-25031	33-25117	
4	3.37	3.37	1.98	1.98	1.98	3.37	3.37	3.37
	33-25044	33-25044	33-25458	33-25458	33-25458	33-535		
3	3.37	3.37	3.37	3.37	3.37	3.37	3.17	2.49
	33-25017	33-25552	33-25101	33-25102	33-25061	33-25523	33-25338	
2	2.49	3.37	3.37	3.37	3.37	3.17	3.37	2.49
	33-8149	33-18037	33-25003	33-25003	33-531	33-18026	33-25009	33-8124
1	2.49	2.49	3.37	3.37	3.37	3.37	2.49	1.98
	33-8123	33-8152	33-25110	33-25109	33-25139	33-25076	33-8133	33-8042

Ringhals 1 Fuel rod diagram for assembly 6478

sid 5

MONTERINGSPROTOKOLL FÖR PATRON: 6478 UTSKRIFTSDATUM: 1988-01-21

DATUM: 880113 OMR: POS: 52 BOX-NR:

	A	B	C	D	E	F	G	H
8	1.98	2.49				1.98	1.98	
	33-8028	33-8081				33-25425	33-8030	
7	2.49	3.37	3.17	3.37	3.37	2.49	2.49	1.98
	33-8089	33-25093	33-25515	33-504	33-25001	33-25310	33-25354	33-8026
6				3.37	3.37	3.17	2.49	1.98
				33-25002	33-25016	33-18027	33-25328	33-25430
5	3.37	3.37					3.37	
	33-25087	33-589					33-25031	
4	3.37				1.98			
	33-25044				33-25453			
3	3.37							
	33-25017							
2	2.49	3.17						
	33-8149	33-18037						
1	2.49	2.49						
	33-8123	33-8152						

Ringhals 1 Fuel rod diagram for assembly 6478

sid 6

UT UR 4020
86 0815

UISKRIFTS DATUM: 1988-01-21

	A	B	C	D	E	F	G	H
8	1.98	2.49	3.37	3.37	3.37	3.37	3.37	1.98
	33-8026	33-8081	33-25093	33-25515	33-25001	33-25316	33-25354	33-8030
7	2.49	3.37	3.37	3.37	3.37	3.37	3.37	2.49
	33-8089	33-25093	33-25515	33-25515	33-25001	33-25316	33-25354	33-2026
6	3.37	3.37	3.37	3.37	3.37	3.37	3.37	3.37
	33-25087	33-25087	33-25087	33-25087	33-25087	33-25087	33-25087	33-25328
5	3.37	3.37	3.37	3.37	3.37	3.37	3.37	3.37
	33-25087	33-25087	33-25087	33-25087	33-25087	33-25087	33-25087	33-25117
4	3.37	3.37	3.37	3.37	3.37	3.37	3.37	3.37
	33-25044	33-25044	33-25044	33-25044	33-25044	33-25044	33-25044	33-535
3	3.37	3.37	3.37	3.37	3.37	3.37	3.37	3.37
	33-25017	33-25017	33-25017	33-25017	33-25017	33-25017	33-25017	33-25338
2	2.49	3.37	3.37	3.37	3.37	3.37	3.37	2.49
	33-8149	33-18037	33-25089	33-25089	33-25089	33-25089	33-25089	33-8124
1	2.49	3.37	3.37	3.37	3.37	3.37	3.37	2.49
	33-8123	33-25110	33-25109	33-25139	33-25076	33-25076	33-25076	33-8042

Ringhals 1 Fuel rod diagram for assembly 8327

8327

NOMMER RINGSPROTOKOLL: 047 RSD4

TOPPL: H474 BOTTL: Z3713 BOX: 73B139 REF:BNRM

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*****
* 1.98  2.49  2.49  2.49  2.49  1.98  1.98  1.38 *
* 2.971 2.966 3.249 3.257 3.261 3.253 2.958 2.956 *
8* 6547  6493 22267 22738 22781 25744  6576  6213 *
*
*
* 2.49  3.37  3.17  3.37  3.37  2.49  2.49  1.98 *
* 2.963 3.265 3.268 3.263 3.266 3.255 3.259 2.960 *
7* 6526 25320 24991  667 25376 22740 22648  6582 *
*
*
* 3.37  3.17BA 3.37  3.37  3.37  3.17BA 2.49  1.98 *
* 3.261 3.241 3.263 3.252 3.252 3.241 3.263 3.257 *
6* 25504 15537 25364 24593 24590 15522 22713 25786 *
*
*
* 3.37  3.37  3.37  1.98  1.98  3.37  3.37  2.49 *
* 3.260 3.268 3.254 3.259 3.257 3.255 3.250 3.264 *
5* 24506  537 24553 25781 25785 24544 24596 22793 *
*
*
* 3.37  3.37  3.37  1.98  3.37  3.37  3.37  2.49 *
* 3.258 3.257 3.258 3.254 3.254 3.254 3.263 3.257 *
4* 24541 24604 24552 47133 25758 24527  547 22729 *
*
*
* 3.37  3.37  3.17  3.37  3.37  3.37  3.17  2.49 *
* 3.252 3.253 3.269 3.260 3.259 3.261 3.271 3.258 *
3* 24621 24620 22783 24509 24587 24526 24995 22839 *
*
*
* 2.49  3.17BA 3.37  3.37  3.37  3.17BA 3.37  2.49 *
* 2.950 3.238 3.254 3.259 3.260 3.241 3.261 2.966 *
2* 6653 15547 24642 24601  542 15475 24574  6522 *
*
*
* 2.49  2.49  3.37  3.37  3.37  3.37  2.49  1.98 *
* 2.950 2.962 3.261 3.254 3.260 3.259 2.964 2.953 *
1* 6617  6513 24629 24659 24676 24623  6482  6588 *
*****

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	A	B	C	D	E	F	G	H
ANRIKN.Z								
NOM	VERKL	ANTAL	KG	KG	KG	UF	FAKTOR	
1.38	1.399	1	2.956	2.605	0.036	0.8814		
1.98	1.997	9	28.124	24.789	0.495	0.8814		
2.49	2.485	16	50.044	44.109	1.094	0.8814		
3.17BA	3.179	4	12.961	11.250	0.356	0.8680		
3.17	3.186	3	9.808	8.644	0.276	0.8814		
3.37	3.364	30	97.754	86.157	2.898	0.8814		
S:A	2.906	63	201.647	177.554	5.155			
INBLANDAD MANGD SD 2.00%								
SPRIDARE NR.: 51003 51286 51260 51262 51259 51281								

GODK. FOR INFrysNING OCH LEVERANS DAT. 08-09-13 SIGN: [Signature]

Ringhals 1 Fuel rod diagram for assembly 8331

8331

MONYERINGS PROTOKOLL PATRON NR 8331

***** 047 X RIFA X

TOPPL: W502 DOTTPL: Z3715 BOX: 738491 REF: HORN

x

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*****
* 1.98 2.49 2.49 2.49 2.49 1.98 1.98 1.38 *
x 2.965 2.964 3.251 3.257 3.257 3.259 2.967 2.965 x
0* 6542 6525 25326 25673 25685 25619 6569 6388 x
*
*
* 2.49 3.37 3.17 3.37 3.37 2.49 2.49 1.98 *
x 2.957 3.261 3.278 3.263 3.262 3.254 3.257 2.966 x
7* 6640 25513 25057 507 25471 25669 23857 6553 x
*
*
x 3.37 3.17BA 3.37 3.37 3.37 3.17BA 2.49 1.98 *
* 3.264 3.241 3.264 3.262 3.261 3.245 3.257 3.258 *
6* 25485 15485 25500 25484 25456 15487 25710 25791 *
*
*
* 3.37 3.37 3.37 1.98 1.98 3.37 3.37 2.49 *
* 3.266 3.263 3.259 3.257 3.258 3.259 3.265 3.250 *
5* 25470 459 25424 25811 25745 25512 25371 23139 *
*
*
* 3.37 3.37 3.37 1.98 3.37 3.37 2.49 *
x 3.265 3.266 3.260 3.262 3.258 3.260 3.260 3.255 x
4* 25430 25447 25402 47052 25742 25407 487 23025 x
*
*
* 3.37 3.37 3.17 3.37 3.37 3.37 3.17 2.49 *
* 3.265 3.266 3.277 3.261 3.265 3.261 3.266 3.263 *
3* 25406 25398 24988 25491 25506 25497 25026 25683 x
*
*
* 2.49 3.17BA 3.37 3.37 3.37 3.17BA 3.37 2.49 *
x 2.958 3.241 3.263 3.262 3.256 3.241 3.266 2.952 x
2* 6635 15463 25565 25482 509 15489 25472 6637 *
*
*
* 2.49 2.49 3.37 3.37 3.37 3.37 2.49 1.98 *
* 2.966 2.969 3.263 3.258 3.261 3.264 2.948 2.965 *
1* 6518 6523 25509 25503 25473 25466 6618 6551 *
*****

```

	A	B	C	D	E	F	G	H
ANRIK.N	1.38	1.98	2.49	3.17	3.37	3.37	3.17	2.49
VERKL	1.394	1.992	2.485	3.179	3.384	3.384	3.384	3.384
ANTAL	1	9	16	4	3	30		
STAVAR								
KG U02	2.965	28.159	50.017	12.968	9.821	97.869		
KG U	2.613	24.817	44.084	11.256	8.654	86.262		
KG U-235	0.036	0.495	1.094	0.357	0.276	2.712		
UFAKTOR	0.8814	0.8814	0.8814	0.8814	0.8814	0.8814		
S:A	2.915	63	201.799	177.690	5.178			
INBLANDAD	MANGD GD 2.00Z							
OPRIDARE NR.:	51030	51299	51277	51297	51289	51273		

Ringhals 1 Fuel rod diagram for assembly 8332

8332

MONTERINGSPROTOKOLL PAIRON BR 0332

047 X RIE4 X

TOPPL: M484 BOTTLPL: M423 BOX: 73B439 REF: BORN

```

*****
* 1.98 2.49 2.49 2.49 2.49 1.98 1.98 1.38 *
* 2.968 2.953 3.267 3.265 3.256 3.258 2.966 2.961 *
8* 8042 8023 30156 30153 30148 30171 8039 8054 *
*
* 2.49 3.37 3.17 3.37 3.37 2.49 2.49 1.98 *
* 2.955 3.251 3.273 3.261 3.255 3.254 3.261 2.963 *
7* 8024 30085 30117 602 30017 30149 30181 8036 *
*
* 3.37 3.17BA 3.37 3.37 3.37 3.17BA 2.49 1.98 *
* 3.261 3.244 3.260 3.254 3.256 3.241 3.261 3.260 *
6* 30067 17013 30026 30102 30062 17001 30132 30180 *
*
* 3.37 3.37 3.37 1.98 1.98 3.37 3.37 2.49 *
* 3.251 3.264 3.253 3.258 3.262 3.252 3.251 3.263 *
5* 30090 605 30084 30167 30161 30083 30108 30128 *
*
* 3.37 3.37 3.37 1.98 3.37 3.37 2.49 *
* 3.250 3.255 3.258 3.261 3.254 3.259 3.257 *
4* 30098 30092 30054 47060 30162 30101 604 30150 *
*
* 3.37 3.37 3.17 3.37 3.37 3.37 3.17 2.49 *
* 3.259 3.252 3.266 3.253 3.251 3.254 3.267 3.270 *
3* 30076 30106 30129 30089 30104 30105 30118 30155 *
*
* 2.49 3.17BA 3.37 3.37 3.37 3.17BA 3.37 2.49 *
* 2.956 3.244 3.252 3.259 3.256 3.247 3.255 2.959 *
2* 8021 17012 30096 30093 609 17005 30097 8028 *
*
* 2.49 2.49 3.37 3.37 3.37 3.37 2.49 1.98 *
* 2.949 2.958 3.257 3.261 3.253 3.254 2.953 2.969 *
1* 8005 8029 30040 30013 30002 30024 8009 8044 *
*****
    
```

ANRIKN. X	ANTAL	KG	KG	KG	UFAKTOR
NOM	VERKL	U02	U	U-235	
1.38	1.407	1	2.961	2.610	0.037
1.98	1.984	9	28.165	24.825	0.493
2.49	2.467	16	50.037	44.103	1.087
3.17BA	3.175	4	12.976	11.263	0.356
3.17	3.186	3	9.806	8.644	0.276
3.37	3.362	30	97.661	86.074	2.890
S:A	2.898	63	201.606	177.519	5.139
INBLANDAD MANGD GD 2.00%					
SPRIDARE NR.: 51048 51293 51288 51256 51272 51292					

SODK. FÖR INFrysNING OCH LEVERANS DAT. SIGN:

Ringhals 1 Fuel rod diagram for assembly 8338

8338

MONTERINGSPROTOKOLL

PATRON NR 0000

047 * R1E4 *

FÖPPL: Z2926

BÖTTPL: M422

BOX: 73B471

REF: HB88

```

*****
* 1.98 2.49 2.49 2.49 2.49 1.98 1.98 1.38 *
* 2.970 2.962 3.255 3.255 3.258 3.256 2.952 2.952 *
8* 6559 6521 32101 32111 32121 32131 6573 6190 *
*
* 2.49 3.37 3.17 3.37 3.37 3.49 3.49 1.98 *
* 2.952 3.266 3.277 3.266 3.261 3.258 3.264 2.963 *
7* 6648 25428 24992 489 25292 22782 22291 6538 *
*
* 3.37 3.17BA 3.37 3.37 3.37 3.17BA 2.49 1.98 *
* 3.253 3.238 3.267 3.257 3.262 3.240 3.257 3.253 *
6* 32071 15457 25433 25316 25295 15465 25686 32074 *
*
* 3.37 3.37 3.37 1.98 1.98 3.37 3.37 2.49 *
* 3.252 3.261 3.261 3.252 3.258 3.261 3.267 3.253 *
5* 32026 499 25349 25770 25743 25354 25408 32023 *
*
* 3.37 3.37 3.37 1.98 3.37 3.37 3.37 2.49 *
* 3.261 3.260 3.271 47004 3.257 3.264 3.266 3.258 *
4* 32076 25502 25415 25772 25386 466 32075 *
*
* 3.37 3.37 3.17 3.37 3.37 3.37 3.17 2.49 *
* 3.261 3.264 3.267 3.257 3.267 3.262 3.277 3.259 *
3* 32025 25372 24968 25448 25414 25365 25808 32024 *
*
* 2.49 3.17BA 3.37 3.37 3.37 3.17BA 3.37 2.49 *
* 2.954 3.237 3.267 3.262 3.262 3.241 3.270 2.951 *
2* 6644 15491 25370 25385 498 15482 25298 6650 *
*
* 2.49 2.49 3.37 3.37 3.37 3.37 2.49 1.98 *
* 2.962 2.949 3.246 3.251 3.253 3.252 2.954 2.966 *
1* 6532 6611 32102 32112 32122 32132 6614 6556 *

```

A B C D E F G H

NOM	ANRIKN. %	ANTAL	VERKL	STAVAR	KG	KG	KG	UFAKTOR
					UO2	U	U-235	
1.38	1.399	1			2.952	2.602	0.036	0.0814
1.98	1.991	9			28.127	24.792	0.495	0.0814
2.49	2.488	16			50.001	44.073	1.094	0.0814
3.17BA	3.179	4			12.956	11.246	0.356	0.0800
3.17	3.186	3			9.821	8.656	0.276	0.0814
3.37	3.383	30			97.030	86.227	2.913	0.0814

SIA 2.915 63 201.687 177.596 5.170

INBLANDAD MÄNGD GD 2.00%

SPRIDARE NR.: 51032 51227 51239 51228 51235 51101

SÖDK. FÖR INFrysNING OCH LEVERANS DAT. SIGN: *[Signature]*

Ringhals 1 Fuel rod diagram for assembly 8341

8341

UNIT: 31143 UNITID: 8593 BOX: 730117 POS: 0000

	1,98	2,49	3,17	3,17	3,17	1,98	1,98	1,98
*	31764	31957	31267	31262	31265	31261	31958	31959
0*	3115	3114	31015	31007	31006	31005	31004	31003
*	3,17	3,17	3,17	3,17	3,17	3,17BA	2,49	1,98
*	31740	31753	31243	31236	31247	31243	31241	31241
2*	3117	31012	31076	102	31049	31071	31003	3117
*	3,17	3,17BA	3,17	3,17	3,17	3,17BA	2,49	1,98
*	31247	31243	31249	31262	31250	31249	31263	31254
6*	31041	10008	31002	31032	31064	10007	31052	31081
*	3,17	3,17	3,17	1,98	1,98	3,17	3,17	2,49
*	31261	31262	31263	31260	31258	31257	31256	31244
2*	31007	704	31001	31004	31026	31010	31066	31083
	138/317/137							
	2665							
	8181	3,17	3,17	1,98	3,17	3,17	2,49	
		31254	31259	31260	31256	31256	31260	
4*		31005	31046	47038	31007	31045	704	31057
*	3,17	3,17	3,17	3,17	3,17	3,17	3,17	2,49
*	31254	31256	31263	31256	31264	31254	31264	31258
3*	31060	31020	31010	31010	31070	31034	31013	31021
*	2,49	3,17BA	3,17	3,17	3,17	3,17BA	3,17	2,49
*	31247	31244	31250	31250	31256	31248	31250	2,957
2*	3109	10009	31062	31073	704	10004	31025	3116
*	2,49	2,49	3,17	3,17	3,17	3,17	2,49	1,98
*	31951	2,946	31247	31244	31256	2,960	2,959	
1*	3103	3107	31049	31002	31072	3104	3123	

INBLANDAD MENGD GD 2.00Z
 OPRIDARE NR.: 51121 51133 51147 51115 51136 51139

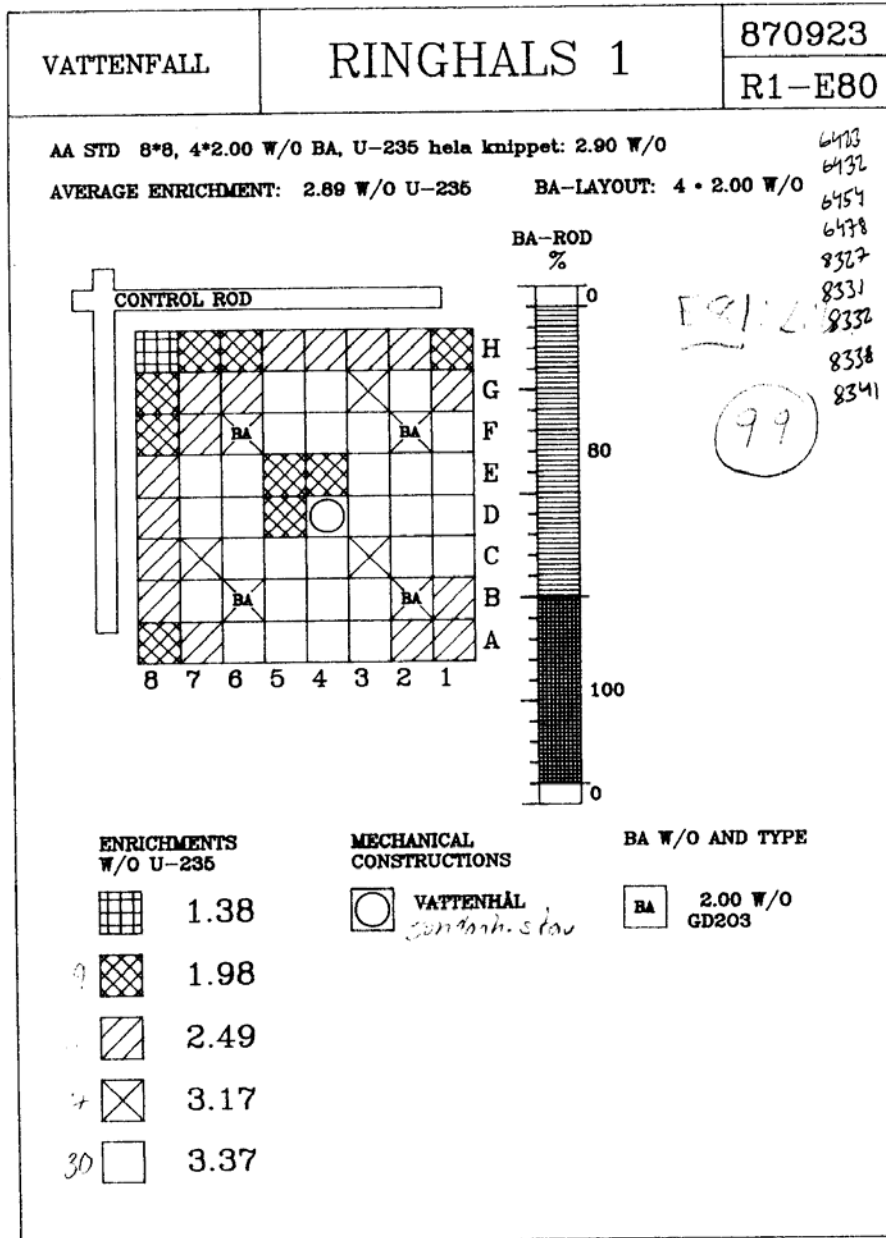
STRAKER STRAVAR I POS 84, 01

Ans	UO ₂	U	U-235
1,38	3 665	3 230	45
1,98	28 135	24 797	493
2,49	50 043	44 108	1 094
3,17 BA	12 984	11 270	358
3,17	10 027	8 838	279
3,37	95 563	84 232	2 830
S:A	200 417	176 475	5 099

51-08-17 / 01

Ringhals 1 Enrichment diagram assemblies 6423, 6432, 6454, 6478, 8327, 8331, 8332, 8338 and 8341

E2
E1
E5A



2.7 Ringhals 2

REACTOR RINGHALS 2

Reactor data	Value
Reactor pressure (MPa) at full power	15.4
Average temperature into core (C) at full power	303.7
Pitch between assemblies in the core (c-c , mm)	215.0
Avg. temperature in the uranium pellets at full power (C)	614
No of BP-rods/ assembly	12
BP material	B2O3
w/o BP	12.5

Cycle data

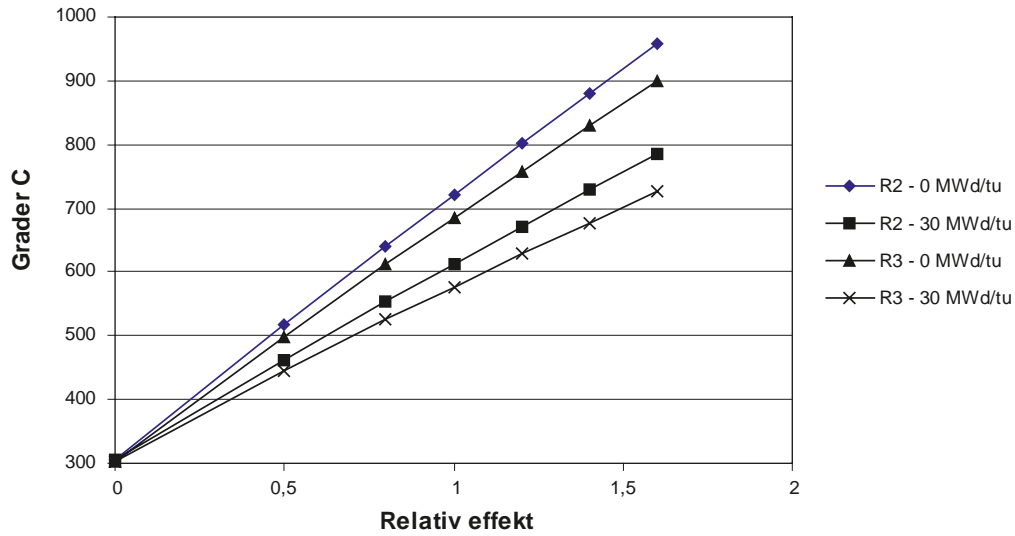
Cycle no	Start	Stopp	Reactor power, MW	Burnup avg/cycle, MWd/tU	No of assemblies in core	Boron content avg., ppm
1	1974-06-19	1977-04-13	2432	16,388	157	
2	1977-07-07	1978-03-31	2432	7,746	157	
3	1978-05-26	1979-04-03	2432	7,615	157	342
4	1979-06-25	1980-04-01	2432	6,856.1	157	530
5	1980-06-18	1981-04-04	2432	8,753.3	157	350
6	1981-06-23	1982-05-06	2432	8,557.8	157	395
7	1982-07-29	1983-04-28	2432	8,287.4	157	308
8	1983-07-28	1984-04-13	2432	7,059.6	157	323
9	1984-07-12	1985-04-04	2432	7,720.9	157	299
10	1985-06-14	1986-04-30	2432	8,261.8	157	340
11	1986-07-02	1987-04-25	2432	7,794.9	157	277
12	1987-06-18	1988-05-12	2432	8,482.5	157	302

Initialdata	C01	C12	C20	C42	D27	D38	E38	E40	F14	F21	F25	F32	G11	G23	I09	I20	I24	I25	
Initial mass Utot (g)	455,789	453,736	454,758	456,159	432,589	434,214	433,593	434,244	436,382	435,939	437,286	436,993	436,180	436,125	437,353	428,147	429,597	433,062	
Initial mass U235 (g)	14,107	14,043	14,075	14,118	14,066	14,119	13,869	13,890	13,953	13,939	13,982	13,972	13,906	13,981	14,007	13,712	13,759	13,870	
Avg.enrichment% U235	3.095	3.095	3.095	3.095	3.252	3.252	3.199	3.199	3.197	3.197	3.197	3.197	3.188	3.206	3.203	3.203	3.203	3.203	
Data after rebuild 1																			
Date of rebuild					810,302													870,122	
No if fuel rods					-1													-2	
No water rods					0													0	
No water holes					0													0	
No of homogeneous rods			1													0			
Mass Utot after rebuild (g)			453,923													423,896			
Mass U235 after rebuild (g)			14,049													13,576			
Data after rebuild 2																			
Date of rebuild																			
No if fuel rods																			
No water rods																			
No water holes																			
No of homogeneous rods																			
Mass Utot after rebuild (g)																			
Mass U235 after rebuild (g)																			
Cycle history burnup/cycle, MWd/tU																			
1	11,247	11,247	11,247	16,565															
2	9,403	9,318	9,377			6,367													
3	7,569	7,390	7,454		9,510	9,331	7,568	7,705											
4					12,889	7,358	8,458	7,249	5,069	4,767	8,307								
5	8,469	8,430			9,267	8,701	9,879	10,655	10,755	6,317	10,749	10,553	6,890						
6				7,619		7,646	8,068	8,730	9,898	10,046	8,316	10,609	10,422	1,0268					
7				8,126	8,010				8,287	8,255	7,980	8,391	7,868	1,0035		8,300	8,245	5,207	
8										6,888			6,943	7,618	6,727	9,010	8,967	4,991	
9			7,642										3,340	7,712	8,950	9,108	9,144	9,803	
10												7,761			9,065	7,895	7,938	8,998	
11												6,629			7,568			7,860	
12				3,329								7,019			7,878				

Axial BU distribution, EOL MWd/kgU

1	20.94	20.89	20.32	18.61	22.79	22.42	17.15	20.16	19.08	21.23	19.89	32.23	19.84	21.05	24.21	19.66	19.59	22.73
2	30.06	29.98	29.21	27.43	33.10	32.17	26.47	28.76	27.51	30.09	28.53	44.46	28.65	29.75	34.16	28.14	28.04	31.98
3	35.08	34.98	34.14	32.76	38.57	37.50	31.56	33.33	32.20	34.78	33.30	50.50	33.54	34.37	39.28	32.81	32.69	36.68
4	37.37	37.25	36.43	35.56	40.89	39.96	34.12	35.32	34.38	36.86	35.53	52.96	35.87	36.41	41.48	35.01	34.87	38.62
5	38.32	38.19	37.41	37.00	41.81	41.05	35.41	36.12	35.38	37.77	36.54	53.88	36.95	37.28	42.38	36.04	35.89	39.33
6	38.68	38.53	37.80	37.73	42.15	41.54	36.14	36.40	35.85	38.15	37.02	54.17	37.46	37.64	42.70	36.52	36.36	39.55
7	38.79	38.63	37.93	38.10	42.29	41.77	36.52	36.49	36.07	38.31	37.25	54.21	37.71	37.78	42.79	36.75	36.58	39.58
8	38.81	38.65	37.97	38.28	42.31	41.89	36.71	36.51	36.17	38.37	37.38	54.18	37.84	37.84	42.79	36.86	36.68	39.54
9	38.81	38.64	37.98	38.38	42.30	41.96	36.83	36.50	36.24	38.39	37.46	54.13	37.93	37.87	42.77	36.92	36.75	39.49
10	38.81	38.64	37.98	38.44	42.29	42.02	36.93	36.50	36.29	38.41	37.52	54.07	37.99	37.89	42.75	36.97	36.79	39.44
11	38.82	38.64	38.00	38.48	42.28	42.08	37.02	36.50	36.33	38.42	37.57	54.01	38.04	37.90	42.73	37.00	36.82	39.40
12	38.83	38.65	38.02	38.52	42.29	42.13	37.11	36.51	36.37	38.42	37.62	53.96	38.10	37.92	42.72	37.04	36.86	39.36
13	38.85	38.67	38.04	38.57	42.30	42.19	37.20	36.53	36.42	38.43	37.68	53.92	38.15	37.94	42.71	37.07	36.89	39.33
14	38.88	38.69	38.07	38.62	42.31	42.24	37.29	36.54	36.46	38.44	37.73	53.88	38.21	37.95	42.70	37.11	36.92	39.30
15	38.91	38.73	38.11	38.68	42.32	42.29	37.38	36.56	36.51	38.45	37.78	53.84	38.26	37.97	42.70	37.15	36.96	39.28
16	38.96	38.77	38.16	38.74	42.33	42.34	37.46	36.57	36.55	38.46	37.82	53.81	38.31	38.00	42.70	37.18	36.99	39.27
17	39.02	38.82	38.22	38.78	42.36	42.38	37.54	36.60	36.59	38.47	37.86	53.78	38.36	38.02	42.70	37.21	37.01	39.26
18	39.08	38.88	38.27	38.76	42.39	42.40	37.58	36.62	36.62	38.46	37.87	53.75	38.38	38.03	42.70	37.21	37.01	39.25
19	39.11	38.91	38.29	38.61	42.40	42.36	37.54	36.62	36.58	38.41	37.82	53.68	38.32	38.00	42.66	37.14	36.94	39.21
20	38.98	38.79	38.15	38.16	42.28	42.14	37.29	36.50	36.39	38.21	37.59	53.45	38.07	37.83	42.48	36.90	36.71	39.06
21	38.40	38.21	37.55	37.07	41.72	41.43	36.53	35.99	35.77	37.62	36.92	52.76	37.35	37.28	41.89	36.22	36.05	38.53
22	36.64	36.46	35.79	34.66	39.96	39.49	34.58	34.45	34.08	35.97	35.15	50.75	35.49	35.69	40.18	34.48	34.32	36.95
23	32.13	31.97	31.35	29.60	35.29	34.68	30.02	30.36	29.89	31.73	30.83	45.37	31.02	31.55	35.66	30.23	30.10	32.73
24	23.16	23.04	22.57	20.74	25.66	25.09	21.35	21.97	21.55	23.08	22.28	33.75	22.30	23.04	26.14	21.84	21.75	23.92

Bränsletemperatur i SIMULATE för Ringhals 2 och 3



Ringhals 2. Temperature program.

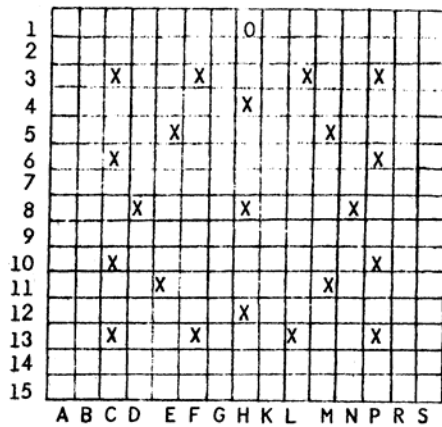
Cykel	HZP Tavg	HFP Tin	Tavg	Tout	100% DT	
R2 Cy01	286.0	286.3	303.7	320.4	34.1	
R2 Cy02	286.0	286.3	303.7	320.4	34.1	
R2 Cy03	286.0	286.3	303.7	320.4	34.1	
R2 Cy04	286.0	286.3	303.7	320.4	34.1	
R2 Cy05	286.0	286.3	303.7	320.4	34.1	
R2 Cy06	286.0	286.3	303.7	320.4	34.1	
R2 Cy07	286.0	286.3	303.7	320.4	34.1	
R2 Cy08	286.0	286.3	303.7	320.4	34.1	
R2 Cy09	286.0	286.3	303.7	320.4	34.1	
R2 Cy10	286.0	286.3	303.7	320.4	34.1	
	286.0	277.1	294.5	311.2	34.1	81% 1986-02-18
R2 Cy11	286.0	277.1	294.5	311.2	34.1	ÅG-probl: 80,7%
R2 Cy12	271.0	271.3	288.7	305.4	34.1	ÅG-probl: 80,7%

Ringhals 2 Fuel rod diagram assembly C42



STATENS VATTENFALLSVERK
VATTENFALL

RINGHALS 2
MONTERINGS PROTOKOLL
PATRON NR: C42



O = Stav av homogen zircaloy
X = Ej möjlig stavposition

Anrikning nominell 3,10
verklig 3,095

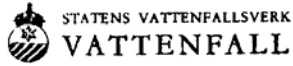
Medelstavvikt (g) UO₂ 2538 U 2236 U-235 69

Uttagna stavar

datum	pos.	UO ₂	vikter i g		U-235
			U	U-235	
810302	H01	2538	2236	69	

Patronvikt efter stavbyte (g)

datum	UO ₂	U	U-235
810302	515142	453923	14049



STATENS VATTENFALLSVERK
VATTENFALL
 RINGHALS 2
 MONTERINGS PROTOKOLL
 PATRON NR: *T20*

1																				
2																				
3			X			X			X			X			X					
4	U							X												
5	U				X								X							
6			X																X	
7																				
8				X				X							X					
9																				
10			X											X					X	
11					X									X						
12								X						X						
13			X			X				X					X					
14																				
15																				
		A	B	C	D	E	F	G	H	K	L	M	N	P	R	S				

O = Stav av homogen zircaloy
 X = Ej möjlig stavposition
 U = Uttagen, ej ersatt stav
 Å = Återinsatt stav

AS: *Skvnr 02503*

Anrikning nominell
 verklig

Medelstavvikt (g) UO₂ U U-235

Uttagna stavar

datum	pos.	UO ₂	vikter i g	U	U-235
<i>870122</i>	<i>AS</i>	<i>2422</i>	<i>2135</i>		<i>68 → Uttagen</i>
<i>870122</i>	<i>E15</i>	<i>2403</i>	<i>2118</i>		<i>68 → Återinsatt</i>

Patronvikt efter stavbyte (g)

datum	UO ₂	U	U-235
	<i>480961</i>	<i>423896</i>	<i>13576</i>

2.8 Ringhals 3

REACTOR RINGHALS 3

Reactor data	Value
Reactor pressure (MPa) at full power	15.5
Average temperature into core (C) at full power	303.3
Pitch between assemblies in the core (c-c, mm)	215.0
Avg. temperature in the uranium pellets at full power (C)	577
No of BP-rods/ assembly	12
BP material	B2O3
w/o BP	12.5

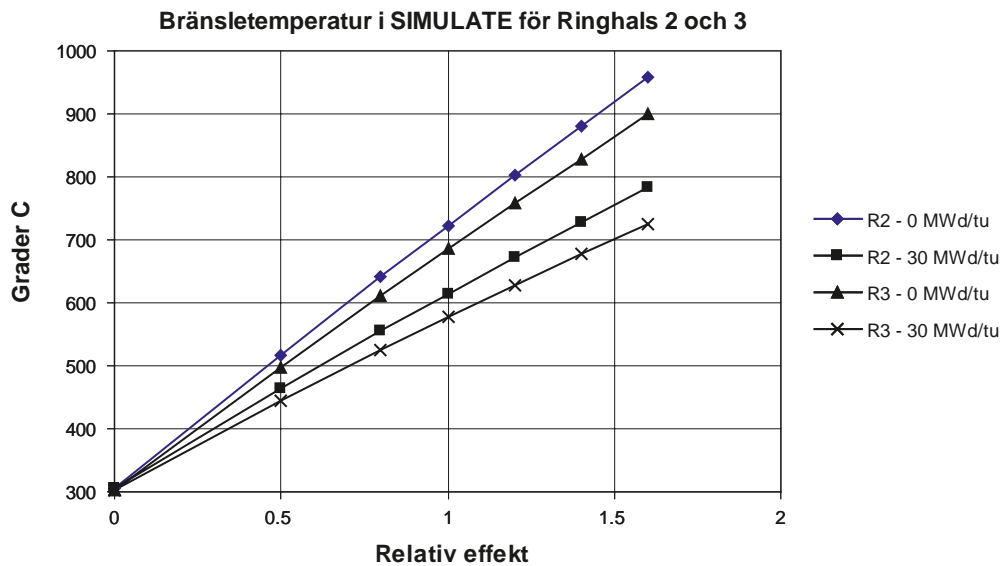
Cycle data

Cycle no	Start	Stopp	Reactor power, MW	Burnup avg/cycle, MWd/tU	No of assemblies in core	Boron content avg., ppm
1A	1980-07-29	1983-06-02	2,775	10,521.50	157	
1B	1983-09-14	1984-05-11	2,775	7,696.40	157	270
2	1984-07-24	1985-05-25	2,775	10,090.60	157	389
3	1985-07-11	1986-05-30	2,775	11,142.30	157	452
4	1986-07-18	1987-06-18	2,775	11,248.00	157	402
5	1987-08-04	1988-07-07	2,775	11,269.10	157	401
6	1988-08-14	1989-06-22	2,775	10,039.40	157	412
7	1989-08-01	1990-06-07	2,775	9,953.70	157	482
8	1990-07-11	1991-06-13	2,775	10,243.10	157	457

Fuel data	Fuel id		0C9	1C2	1C5	2C2	3C1	3C4	3C5	3C9	4C4	4C7	0E2	0E6	1E5	5F2
	2A5	5A3														
Fuel type	W17x17	W17x17	W17x17	W17x17	W17x17	W17x17	W17x17	W17x17	W17x17	W17x17	W17x17	W17x17	W17x17	W17x17	W17x17	AA17x17
No of fuel rods	264	264	264	264	264	264	264	264	264	264	264	264	264	264	264	264
Rod pitch (mm)	12.6	12.6	12.6	12.6	12.6	12.6	12.6	12.6	12.6	12.6	12.6	12.6	12.6	12.6	12.6	12.6
Rod diameter (mm)	9.5	9.5	9.5	9.5	9.5	9.5	9.5	9.5	9.5	9.5	9.5	9.5	9.5	9.5	9.5	9.5
Clad thickness (mm)	0.571	0.571	0.571	0.571	0.571	0.571	0.571	0.571	0.571	0.571	0.571	0.571	0.572	0.572	0.572	0.572
Pellet diameter (mm)	8.191	8.191	8.191	8.191	8.191	8.191	8.191	8.191	8.191	8.191	8.191	8.191	8.191	8.191	8.191	8.191
Cladding material	Zr4	Zr4	Zr4	Zr4	Zr4	Zr4	Zr4	Zr4	Zr4	Zr4	Zr4	Zr4	Zr4	Zr4	Zr4	Zr2
Active length (mm)	3,658	3,658	3,658	3,658	3,658	3,658	3,658	3,658	3,658	3,658	3,658	3,658	3,658	3,658	3,658	3,658
Density UO2 (g/cc)	10.45	10.45	10.45	10.45	10.45	10.45	10.45	10.45	10.45	10.45	10.45	10.45	10.45	10.45	10.45	10.45
Density incl. porosity, dishing etc	10.31	10.3	10.21	10.24	10.22	10.25	10.23	10.22	10.24	10.25	10.24	10.23	10.35	10.31	10.35	10.32
No of guide tubes	24	24	24	24	24	24	24	24	24	24	24	24	24	24	24	24
Material in guide tubes	Zr4	Zr4	Zr4	Zr4	Zr4	Zr4	Zr4	Zr4	Zr4	Zr4	Zr4	Zr4	Zr4	Zr4	Zr4	Zr4
Outer diameter guide tube (mm)	12.24	12.24	12.24	12.24	12.24	12.24	12.24	12.24	12.24	12.24	12.24	12.24	12.24	12.24	12.24	12.09
Cladding thickness guide tube	0.406	0.406	0.406	0.406	0.406	0.406	0.406	0.406	0.406	0.406	0.406	0.406	0.406	0.406	0.406	0.455
No of instrument tubes	1	1	1	1	1	1	1	1	1	1	1	1	1	1	1	1
Material in instrument tubes	Zr4	Zr4	Zr4	Zr4	Zr4	Zr4	Zr4	Zr4	Zr4	Zr4	Zr4	Zr4	Zr4	Zr4	Zr4	Zr4
Outer diameter instrument tubes (mm)	12.24	12.24	12.24	12.24	12.24	12.24	12.24	12.24	12.24	12.24	12.24	12.24	12.24	12.24	12.24	12.24
Cladding thickness instrument tubes (mm)	0.406	0.406	0.406	0.406	0.406	0.406	0.406	0.406	0.406	0.406	0.406	0.406	0.406	0.406	0.406	0.406
Burnable poison rods?	No	No	Yes, 12	No	Yes, 12	No	No	Yes, 12	Yes, 12	No	No	Yes, 12	No	No	No	No
No of spacers	8	8	8	8	8	8	8	8	8	8	8	8	8	8	8	8
Spacer material	Inc 718	Inc 718	Inc 718	Inc 718	Inc 718	Inc 718	Inc 718	Inc 718	Inc 718	Inc 718	Inc 718	Inc 718	Inc 718	Inc 718	Inc 718	Zr4
Mass of spacer (g)	753	753	753	753	753	753	753	753	753	753	753	753	753	753	753	1,350

Axial BU distribution, EOL MWd/tU

1	9.65	8.66	21.14	18.08	21.16	20.09	20.09	21.14	21.12	20.09	18.08	21.12	24.59	20.54	18.93	28.28
2	14.88	14.07	30.64	26.61	30.67	29.19	29.19	30.64	30.60	29.18	26.61	30.60	34.69	29.34	27.56	39.72
3	18.29	17.62	36.17	31.54	36.21	34.45	34.45	36.18	36.13	34.44	31.54	36.12	40.17	34.27	32.45	45.78
4	20.21	19.66	38.93	33.88	38.97	37.03	37.02	38.94	38.88	37.01	33.89	38.87	42.65	36.57	34.84	48.49
5	21.23	20.90	40.25	34.93	40.29	38.21	38.21	40.26	40.19	38.20	34.94	40.19	43.72	37.60	36.02	49.63
6	21.73	21.42	40.85	35.37	40.89	38.72	38.72	40.86	40.78	38.71	35.38	40.78	44.17	38.05	36.56	50.09
7	21.95	21.62	41.10	35.58	41.14	38.94	38.94	41.11	41.03	38.92	35.59	41.03	44.35	38.25	36.84	50.27
8	22.03	21.67	41.20	35.66	41.24	39.02	39.02	41.21	41.13	39.01	35.67	41.13	44.41	38.35	36.99	50.34
9	22.04	21.66	41.25	35.69	41.28	39.06	39.05	41.25	41.17	39.04	35.71	41.17	44.44	38.41	37.07	50.37
10	22.03	21.63	41.28	35.72	41.31	39.08	39.07	41.28	41.20	39.06	35.74	41.19	44.44	38.45	37.14	50.38
11	22.01	21.61	41.30	35.75	41.34	39.11	39.10	41.31	41.22	39.09	35.77	41.22	44.44	38.48	37.19	50.39
12	22.01	21.61	41.33	35.78	41.37	39.15	39.14	41.34	41.25	39.13	35.80	41.25	44.43	38.52	37.25	50.40
13	22.02	21.62	41.37	35.82	41.41	39.20	39.20	41.38	41.29	39.18	35.83	41.29	44.42	38.55	37.30	50.41
14	22.05	21.65	41.42	35.86	41.46	39.27	39.26	41.42	41.34	39.25	35.87	41.34	44.41	38.58	37.35	50.42
15	22.10	21.69	41.47	35.90	41.51	39.35	39.34	41.48	41.39	39.33	35.92	41.39	44.40	38.62	37.40	50.43
16	22.15	21.75	41.53	35.94	41.57	39.44	39.43	41.54	41.45	39.42	35.96	41.45	44.39	38.66	37.45	50.44
17	22.20	21.81	41.59	35.99	41.63	39.54	39.53	41.59	41.50	39.52	36.01	41.50	44.38	38.70	37.50	50.46
18	22.22	21.84	41.61	36.02	41.66	39.63	39.62	41.62	41.53	39.61	36.04	41.53	44.36	38.73	37.52	50.46
19	22.15	21.78	41.55	35.99	41.61	39.65	39.64	41.56	41.47	39.63	36.01	41.47	44.29	38.71	37.48	50.40
20	21.85	21.52	41.25	35.78	41.31	39.47	39.46	41.26	41.17	39.45	35.80	41.17	44.07	38.51	37.26	50.11
21	21.11	20.82	40.37	35.09	40.44	38.74	38.73	40.38	40.29	38.72	35.11	40.29	43.37	37.83	36.57	49.28
22	19.49	19.26	38.18	33.25	38.25	36.74	36.73	38.18	38.10	36.72	33.27	38.10	41.42	35.99	34.71	47.05
23	16.25	16.09	33.14	28.87	33.20	31.96	31.95	33.14	33.07	31.94	28.89	33.07	36.49	31.48	30.26	41.50
24	10.92	10.83	23.69	20.55	23.73	22.84	22.83	23.69	23.64	22.82	20.57	23.64	26.58	22.68	21.67	30.32



Ringhals 3. Temperature program.

Cykel	HZP Tavg	Tin	HFP Tavg	Tout	100% DT	
R3 C01A	291.7	285.3	303.3	321.8	36.5	
R3 C01B	291.7	285.3	303.3	321.8	36.5	
R3 C02	291.7	285.3	303.3	321.8	36.5	
R3 C03	291.7	285.3	303.3	321.8	36.5	
R3 C04	291.7	285.3	303.3	321.8	36.5	
R3 C05	291.7	285.3	303.3	321.8	36.5	
R3 C06	291.7	285.3	303.3	321.8	36.5	
	291.7	273.3	292.0	309.8	36.5	Max 88%
R3 C07	291.7	273.3	292.0	309.8	36.5	Max 88%
R3 C08	291.7	273.3	291.9	309.8	36.5	Maz 88%

Ringhals 3 – Fuel rod diagram assembly 3C4

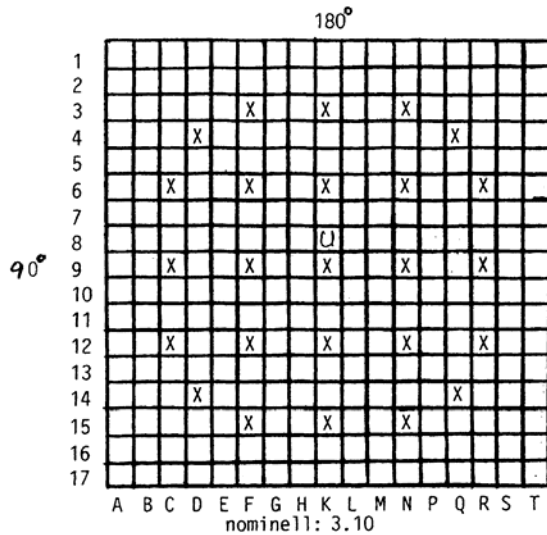
Vattenfall

RINGHALSVERKET

RINGHALS 3

MONTERINGSPROTOKOLL

PATRON NR: 3C4



(Patronen sedd ovanifrån)

0 = Stav av homogen zircaloy

X = Ej möjlig stavposition

U = uttagen stav

Anrikning

verklig: 3.101

Medelstavvikt (g) UO₂ 1968 U 1734 U-235 54

Uttagna stavar

(vikter i g)

datum pos. UO₂ U U-235

870827 K8 1968 1734 54 Anm. Inga dumstavar insatta.

Patronvikt efter stavbyte (g)

datum UO₂ U U-235

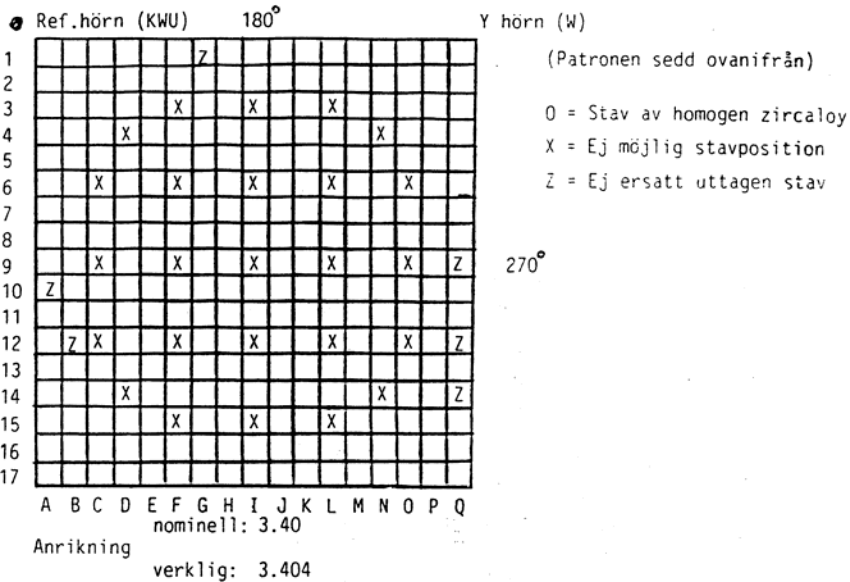
870827 517592 456170 14146

B73

Vattenfall

RINGHALSVERKET

RINGHALS 3
 MONTERINGSPROTOKOLL
 PATRON NR: 5F2



Uttagna stavar		(vikter i g)		U-235	stav nr.
datum	pos.	UO ₂	U		
911123	B12	1984	1749	60	096-33846
911123	Q14	1988	1752	60	096-40003
911123	Q12	1987	1751	60	096-40015
920404	A10	1984	1749	60	096-33831
920404	G1	1988	1752	60	096-33949
920404	Q9	1988	1752	60	096-40064

Patronvikt efter stavbyte (g)			
datum	UO ₂	U	U-235
911123	518493	456996	15654
920404	512533	451743	15474

2.9 Barsebäck 1

REACTOR BARSEBÄCK 1

Parameter	Value
Reactor pressure (MPa) at full power	7
Inlet temperature into core (C) at full power	260
Outlet temperature into core (C) at full power	286
Pitch between assemblies in the core (c-c, mmm)	153
Avg. temperature in the uranium pellets at full power (C)	650
Avg. temperature in the cladding at full power (C)	290
Temperature in the boxwall at full power (C)	286

Cycle data

Cycle no	Start	Stop	Reactor power MW	Burnup avg/cycle MWd/tU	No of assemblies in core
1	1975-05-15	1977-07-16	1,800	11,140	444
2	1977-10-10	1978-05-08	1,800	3,878	444
3	1978-05-30	1979-04-13	1,800	5,738	444
4	1979-09-05	1980-09-03	1,800	6,441	444
5	1980-10-26	1981-06-22	1,800	4,919	444
6	1981-07-19	1982-06-30	1,800	6,864	444
7	1982-08-14	1983-07-06	1,800	6,745	444
8	1983-08-06	1984-06-28	1,800	6,666	444
9	1984-07-26	1985-08-07	1,800	7,572	444
10	1985-09-02	1986-07-17	1,800	6,653	444
11	1986-08-18	1987-07-01	1,800	6,940	444
12	1987-07-29	1988-09-17	1,800	8,750	444

Fuel data	Fuel id 2014	2018	2048	2074	2118	9329	10288
Fuel type	AA8x8	AA8x8	AA8x8	AA8x8	AA8x8	AA8x8	AA8x8
No of fuel rods	63	63	63	63	63	63	63
Rod pitch normal rods (mm)	16.3	16.3	16.3	16.3	16.3	16.3	16.3
Rod pitch normal – corner rods (mm)	16.05	16.05	16.05	16.05	16.05	16.05	16.05
Rod pitch corner – corner rods (mm)	15.8	15.8	15.8	15.8	15.8	15.8	15.8
Rod diameter normal rod (mm)	12.25	12.25	12.25	12.25	12.25	12.25	12.25
Clad thickness normal rod (mm)	0.8	0.8	0.8	0.8	0.8	0.8	0.77
Pellet diameter normal rod (mm)	10.46	10.46	10.46	10.46	10.46	10.44	10.44
No of normal rods	51	51	51	51	51	51	51
Rod diameter corner rod (mm)	11.75	11.75	11.75	11.75	11.75	11.75	11.75
Clad thickness corner rod (mm)	0.8	0.8	0.8	0.8	0.8	0.8	0.77
Pellet diameter corner rod (mm)	9.96	9.96	9.96	9.96	9.96	9.94	9.94
No of corner rods	12	12	12	12	12	12	12
Active length (mm)	3,712	3,712	3,712	3,712	3,712	3,712	3,712
Density UO2 (g/cc)	10.5	10.5	10.5	10.5	10.5	10.5	10.5
Porsity,dishing etc (%)	1.49	1.5	1.56	1.61	1.63	1.71	1.50
No of spacer rods	1	1	1	1	1	1	1
Material in spacer rods	Zr2	Zr2	Zr2	Zr2	Zr2	Zr2	Zr2
Outer diameter spacer rods (mm)	12.25	12.25	12.25	12.25	12.25	12.25	12.25
Cladding thickness spacer rod							
No of water rods							
Material in water rods							
Outer diameter water rods (mm)							
Inner diameter water rods (mm)							
No of BA rods						5	5
% Gd2O3						2.55	2.55
Rel poison						1	1
No of spacers	6	6	6	6	6	6	6
Spacer material	Inconel	Inconel	Inconel	Inconel	Inconel	Inconel	Inconel
Mass of spacer (g)	135	135	135	135	135	135	135
Box material	Zr4	Zr4	Zr4	Zr4	Zr4	Zr4	Zr4
Box outer measure square (mm)	139	139	139	139	139	139	139
Box wall thickness (mm)	2.3	2.3	2.3	2.3	2.3	2.3	2.3

Initial data	2014	2018	2048	2074	2118	9329	10288
Initial mass Utot (g)	179,878	179,852	179,741	179,657	179,607	178,771	179,159
Initial mass U235 (g)	4,183	4,183	4,190	4,179	4,485	5,223	5,291
Avg.enrichment% U235	2.320	2.320	2.320	2.320	2.500	2.920	2.950
Fuel rod diagram	X	X	X	X	X	X	X
Enrichment diagram						X	X
Data after rebuild 1							
Date of rebuild	1977	1977	1977	1977	1977		
No of fuel rods	63	63	63	63	63		
No spacer rods	1	1	1	1	1		
No water rods							
No of water holes							
Mass Utot after rebuild (g)							
Mass U235 after rebuild (g)							
Data after rebuild 2							
Date of rebuild							
No of fuel rods							
No spacer rods							
No water rods							
No of water holes							
Mass Utot after rebuild (g)							
Mass U235 after rebuild (g)							
Cycle history burnup/cycle, MWd/tU							
1	13,200	12,700	11,800	11,500	12,000		
2		1,900	4,500	4,100	4,500		
3	6,448	3,081	6,259				
4		2,924		7,323			
5							
6						4,154	
7							
8						8,219	
9						9,508	9,726
10						7,307	7,714
11						6,924	8,147
12						9,136	9,593
Axial BU distribution, EOL MWd/tUNode no (top=25)							
1						30,200	18,300
2						31,900	29,600
3						37,700	35,100
4						40,300	37,300
5						42,600	39,200
6						43,900	40,000
7						44,700	40,200
8						44,700	39,700
9						45,800	40,300
10						46,200	40,400
11						46,500	40,300
12						46,200	39,600
13						46,700	39,900
14						46,800	37,700
15						46,500	39,200
16						45,800	38,200
17						46,000	38,100
18						45,700	37,600
19						44,900	36,700
20						43,500	35,100
21						42,400	33,900
22						39,900	31,600
23						35,700	28,000
24						22,900	23,300
25						22,400	17,600

Void history Node no (top=25)	2014	2018	2048	2074	2118	9329	10288
1						0.00	0.00
2						0.00	0.00
3						0.00	0.00
4						0.03	0.05
5						0.08	0.12
6						0.15	0.19
7						0.21	0.26
8						0.28	0.31
9						0.33	0.37
10						0.38	0.42
11						0.43	0.46
12						0.47	0.50
13						0.51	0.54
14						0.54	0.57
15						0.57	0.60
16						0.60	0.63
17						0.63	0.65
18						0.65	0.67
19						0.67	0.68
20						0.68	0.70
21						0.70	0.71
22						0.71	0.73
23						0.72	0.73
24						0.73	0.74
25						0.74	0.75
Density history Node no (top=25)							
1							
2							
3							
4							
5							
6							
7							
8							
9							
10							
11							
12							
13							
14							
15							
16							
17							
18							
19							
20							
21							
22							
23							
24							
25							

Average density	0.41	0.41	0.41	0.41	0.41	0.39	0.40
Control rod history							
Node no (top=25)							
1							
2							
3							
4							
5							
6							
7							
8							
9							
10							
11							
12							
13							
14							
15							
16							
17							
18							
19							
20							
21							
22							
23							
24							
25							

Barsebäck 1 Fuel rod diagram assembly 2014

M O N T E R I N G S P R O T O K O L L

 B A R S E B Ä C K I

PATRON NR.2014.

DATUM:24-09-74

REF.HÖRN

*	1.85	1.85	1.35	1.85	1.17	1.17	1.17	1.17	*
*	2.988	2.996	3.291	3.303	3.302	3.307	2.996	2.994	*
*	7397	7453	19252	19236	16059	16082	5301	5749	*
*									*
*	1.85	2.50	2.50	2.50	1.85	1.85	1.17	1.17	*
*	2.988	3.296	3.293	3.283	3.294	3.299	3.311	3.005	*
*	7467	27002	27080	782	19262	19253	16013	5733	*
*									*
*	2.50	2.50	3.05	2.50	2.50	1.85	1.85	1.17	*
*	3.304	3.292	3.309	3.297	3.308	3.293	3.301	3.299	*
*	27017	27106	24660	27015	27129	19249	19240	16001	*
*									*
*	3.05	3.05	3.05	1.85	1.85	2.50	1.85	1.17	*
*	3.300	3.296	3.297	3.298	3.304	3.297	3.303	3.302	*
*	24557	1459	24534	19263	19223	27130	19244	16020	*
*									*
*	3.05	3.05	3.05	1.85	1.85	2.50	2.50	1.85	*
*	3.296	3.302	3.297	3.300	3.298	3.294	3.301	3.289	*
*	24542	24539	24529	19248	19247	26980	872	19246	*
*									*
*	3.05	3.05	3.05	3.05	3.05	3.05	2.50	1.85	*
*	3.306	3.304	3.297	3.306	3.301	3.299	3.293	3.303	*
*	24663	24651	24533	24623	24550	24632	27004	19132	*
*									*
*	2.50	3.05	3.05	3.05	3.05	2.50	2.50	1.85	*
*	2.994	3.291	3.304	3.302	3.291	3.295	3.296	2.992	*
*	10303	24577	24886	24733	1373	27090	26996	7464	*
*									*
*	1.85	2.50	3.05	3.05	3.05	2.50	1.85	1.85	*
*	2.990	2.993	3.305	3.298	3.310	3.292	2.992	2.992	*
*	7471	10275	24900	24382	24885	26042	7474	7424	*

ANRIKN. %	ANTAL	KG	KG	KG	
NOH VERKL.	STAVAR	U02	U	U-235	
1.17	1.176	8	25.516	22.480	0.264
1.35	1.850	20	63.314	56.220	1.040
2.50	2.499	16	52.133	45.929	1.148
3.05	3.069	20	66.011	58.156	1.785
1.85	1.850	19	60.514	53.313	0.986
S:A	2.318	64	207.474	182.785	4.237
	2.525	63	204.174	179.878	4.183

TOPPL.NR.: Z914
 BOTTPL.NR.: K367

SPRIDARE NR.: 834 2598 2428 2633 1015 1011 URSPR.LAND FÖR URAN: US,
 BOXNR.: 1977
 Avsant

21 OKT 1974

6/12

Barsebäck 1 Fuel rod diagram assembly 2018

MONTERINGS PROTOKOLL

 B A R S E B Ä C K I

PATRON NR.2018.

DATUM:25-09-74

REF. HORN

*	1.35	1.35	1.35	1.35	1.17	1.17	1.17	1.17	*
*	2.978	2.992	3.293	3.291	3.310	3.298	3.303	2.995	*
*	7359	7384	19168	19086	15940	15931	5327	5828	*
*									*
*	1.85	2.50	2.50	2.50	1.85	1.85	1.17	1.17	*
*	2.989	3.288	3.286	3.285	3.294	3.306	3.300	2.993	*
*	7440	26969	26987	792	19196	19225	15938	5757	*
*									*
*	2.50	2.50	3.05	2.50	2.50	1.85	1.85	1.17	*
*	3.289	3.299	3.306	3.298	3.305	3.289	3.298	3.298	*
*	26985	27097	24559	26981	26944	19126	19264	15937	*
*									*
*	3.05	3.05	3.05	1.85	1.85	2.50	1.85	1.17	*
*	3.309	3.293	3.304	3.294	3.294	3.296	3.295	3.299	*
*	24619	1393	24618	19123	19204	26935	19109	15951	*
*									*
*	3.05	3.05	3.05	1.85	1.85	2.50	2.50	1.85	*
*	3.305	3.303	3.306	3.298	3.293	3.301	3.296	3.302	*
*	24664	24616	24603	19118	19084	27000	844	19116	*
*									*
*	3.05	3.05	3.05	3.05	3.05	3.05	2.50	1.85	*
*	3.299	3.300	3.298	3.305	3.306	3.303	3.286	3.302	*
*	24568	24569	24585	24533	24606	24560	26986	19087	*
*									*
*	2.50	3.05	3.05	3.05	3.05	2.50	2.50	1.85	*
*	2.983	3.305	3.306	3.306	3.294	3.294	3.302	2.990	*
*	10341	24600	24631	24602	1402	26948	27128	7406	*
*									*
*	1.85	2.50	3.05	3.05	3.05	2.50	1.85	1.85	*
*	2.988	3.000	3.303	3.305	3.306	3.301	2.996	2.984	*
*	7376	10293	24615	24614	24601	26957	7356	7408	*

ANRIK.N.%	ANTAL	KG	KG	KG	
NOM VERKL.	STAVAR	U02	U	U-235	
1.17	1.176	8	25.496	22.462	0.264
1.85	1.850	20	63.771	56.182	1.039
2.50	2.499	16	52.109	45.908	1.147
3.05	3.069	20	66.067	58.205	1.786
1.85	1.850	19	60.473	53.277	0.986
S:A	2.318	64	207.443	182.757	4.236
	2.325	63	204.145	179.852	4.183

SPRIDARE NR.: 2442 2426 1497 2433 1546 1022 URSPR.LAND FÖR URAN: US
 BOXNR.: 1843

175669
 Avsant
 21 OKT 1974 6/2

Barsebäck 1 Fuel rod diagram assembly 2048

MONTERINGS PROTOKOLL

 B A R S E B Ä C K I

PATRON NR. 2048.

DATUM: 11-10-74

REF. HÖRN

*	1.85	1.85	1.85	1.85	1.17	1.17	1.17	1.17	*
*	2.986	2.982	3.296	3.298	3.320	3.310	3.004	2.994	*
*	8074	8123	19379	19369	16277	16273	5730	5735	*
*									*
*	1.85	2.50	2.50	2.50	1.85	1.85	1.17	1.17	*
*	2.983	3.299	3.301	3.303	3.292	3.294	3.314	2.993	*
*	7987	26886	26850	703	19427	19460	16276	5739	*
*									*
*	2.50	2.50	3.05	2.50	2.50	1.85	1.85	1.17	*
*	3.292	3.291	3.293	3.299	3.295	3.298	3.294	3.316	*
*	26839	26845	24973	26729	26739	19478	19390	16335	*
*									*
*	3.05	3.05	3.05	1.85	1.85	2.50	1.85	1.17	*
*	3.297	3.301	3.288	3.297	3.307	3.293	3.295	3.306	*
*	25104	1638	25120	19444	19350	26751	19498	16255	*
*									*
*	3.05	3.05	3.05	1.85	1.85	2.50	2.50	1.85	*
*	3.291	3.303	3.290	3.290	3.295	3.284	3.291	3.293	*
*	25099	25095	25118	19318	19472	26783	730	19389	*
*									*
*	3.05	3.05	3.05	3.05	3.05	3.05	2.50	1.85	*
*	3.291	3.298	3.301	3.297	3.301	3.293	3.291	3.296	*
*	24940	25124	25121	24963	25009	25080	26757	19480	*
*									*
*	2.50	3.05	3.05	3.05	3.05	2.50	2.50	1.85	*
*	2.998	3.298	3.293	3.296	3.300	3.282	3.301	2.982	*
*	10037	25107	24983	25065	1669	26785	26747	7978	*
*									*
*	1.85	2.50	3.05	3.05	3.05	2.50	1.85	1.35	*
*	2.990	2.994	3.287	3.294	3.291	3.291	2.982	2.988	*
*	7992	10039	24953	24931	25090	26760	7980	7982	*

ANRIKN. %	ANTAL	KG	KG	KG	
NOM VERKL.	STAVAR	U02	U	U-235	
1.17	1.176	8	25.557	22.516	0.265
1.85	1.850	20	63.743	56.158	1.039
2.50	2.501	16	52.105	45.905	1.148
3.05	3.086	20	65.903	58.061	1.792
1.85	1.850	19	60.453	53.259	0.965
S:A	2.323	64	207.308	182.640	4.244
	2.331	63	204.018	179.741	4.190

TOPPL.NR.: 2922
 BOTTPL.NR.: K802

SPRIDARE NR.: 2552 1951 2344 2279 2342 2553 URSPR. LAND FÖR UFAN: US,
 BOXNR.: 1903

Avsänd
 20 Okt 1974
 Sign..... 616
 631

Barsebäck 1 Fuel rod diagram assembly 2074

MONTERINGS PROTOKOLL

 B A R S E B Ä C K I

PATRON NR. 2074. DATUM: 31-10-74 REF. HORN

*

*	1.85	1.85	1.85	1.85	1.17	1.17	1.17	1.17	*
*	2.980	2.984	3.311	3.301	3.299	3.305	2.985	2.980	*
*	8292	8293	20761	20759	16620	16629	6224	6274	*
*									*
*	1.85	2.50	2.50	2.50	1.85	1.85	1.17	1.17	*
*	2.981	3.288	3.283	3.297	3.305	3.304	3.305	2.988	*
*	8249	29056	29040	1130	20729	20757	16643	6335	*
*									*
*	2.50	2.50	3.05	2.50	2.50	1.85	1.85	1.17	*
*	3.284	3.291	3.293	3.279	3.310	3.305	3.306	3.308	*
*	29052	29053	33205	29004	28981	20758	20781	16735	*
*									*
*	3.05	3.05	3.05	1.85	1.85	2.50	1.85	1.17	*
*	3.295	3.301	3.279	3.309	3.301	3.282	3.314	3.310	*
*	33204	164	33162	20769	20792	29113	20636	16616	*
*									*
*	3.05	3.05	3.05	1.85	1.85	2.50	2.50	1.85	*
*	3.302	3.300	3.284	3.302	3.303	3.285	3.306	3.312	*
*	33073	33031	33044	20637	20699	28999	1265	20742	*
*									*
*	3.05	3.05	3.05	3.05	3.05	3.05	2.50	1.85	*
*	3.297	3.294	3.291	3.282	3.279	3.295	3.293	3.307	*
*	33035	33034	33029	33046	33089	33159	29034	20771	*
*									*
*	2.50	3.05	3.05	3.05	3.05	2.50	2.50	1.85	*
*	2.995	3.289	3.287	3.286	3.294	3.286	3.298	2.993	*
*	10651	33030	33091	33090	169	29131	29161	8269	*
*									*
*	1.85	2.50	3.05	3.05	3.05	2.50	1.85	1.85	*
*	2.989	2.996	3.278	3.286	3.285	3.286	2.982	2.991	*
*	8299	10724	33038	33087	33086	29002	8213	8285	*

ANRIKN.	%	ANTAL	KG	KG	KG	
NO4	VERKL.	STAVAR	U02	U	U-235	
1.17	1.179	3	25.430	27.448	0.265	TOPPL.NR.: 2505
1.35	1.350	20	63.335	56.283	1.041	
2.50	2.501	16	52.064	45.868	1.147	BOTTPL.NR.: K1050
3.05	3.071	20	65.797	57.967	1.780	
7.85	7.85	79	20.615	85.334	2.179	
S:A	2.319	64	207.226	182.500	4.233	
	2.319	64	207.226	182.500	4.233	

SPRIDARE NR.: 814 371 303 772 804 470 URSPR. LAND FÖR URAN: USA
 BOXNR.: 1662

Barsebäck 1 Fuel rod diagram assembly 2118

MONTERINGS PROTOKOLL

 B A R S E B Ä C K I

PATRON NR. 2118.

DATUM: 04-11-74

REF. HÖRN

```

*****
* 1.85 1.85 1.35 1.85 1.17 1.17 1.17 1.17 *
* 2.983 2.986 3.303 3.296 3.304 3.306 2.985 2.978 *
* 8353 8361 20773 20772 16731 16740 6262 6209 *
*
*
* 1.85 2.50 2.50 2.50 1.85 1.85 1.17 1.17 *
* 2.977 3.293 3.293 3.307 3.303 3.303 3.310 2.981 *
* 8313 29099 29091 1232 20715 20714 16631 6126 *
*
*
* 2.50 2.50 3.05 2.50 2.50 1.85 1.85 1.17 *
* 3.292 3.287 3.293 3.295 3.281 3.305 3.312 3.303 *
* 29062 29117 33190 29084 29063 20812 20691 16715 *
*
*
* 3.05 3.05 3.05 1.85 1.85 2.50 1.85 1.17 *
* 3.292 3.301 3.286 3.306 3.297 3.296 3.299 3.297 *
* 33189 137 33143 20780 20798 29042 20836 16664 *
*
*
* 3.05 3.05 3.05 1.85 1.85 2.50 2.50 1.85 *
* 3.292 3.279 3.287 3.311 3.306 3.301 3.307 3.305 *
* 33181 33065 33156 20628 20749 29074 1323 20777 *
*
*
* 3.05 3.05 3.05 3.05 3.05 3.05 2.50 1.85 *
* 3.293 3.286 3.275 3.295 3.286 3.283 3.301 3.311 *
* 33158 33078 33122 33157 33072 33150 29150 20627 *
*
*
* 2.50 3.05 3.05 3.05 3.05 2.50 2.50 1.85 *
* 2.990 3.281 3.287 3.285 3.293 3.297 3.291 2.992 *
* 10671 33054 33136 33041 180 29069 29073 8177 *
*
*
* 1.85 2.50 3.05 3.05 3.05 2.50 1.85 1.85 *
* 2.983 2.995 3.286 3.288 3.297 3.292 2.991 2.995 *
* 8254 10668 33109 33045 33108 29072 8236 8199 *
  
```

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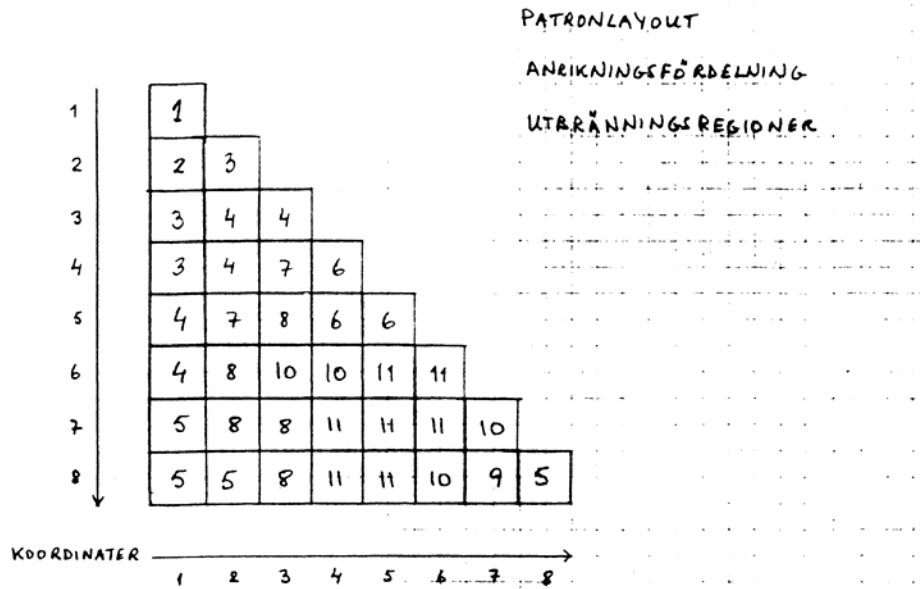
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ANRIKN. % ANTAL KG KG KG
NOM VEFKL. STAVAR UO2 U U-235
1.17 1.179 8 25.464 22.434 2.264 TOPPL.NR.: 2313
1.85 1.350 20 63.869 56.269 1.041
2.50 2.501 16 52.128 45.925 1.148 BOTTL.NR.: K1093
3.05 3.071 20 65.730 57.952 1.780

S:A 2.319 64 207.241 182.580 4.233
  
```

SPRIDARE NR.: 2052 1451 1468 2047 1464 1486 HRSPR. LAND FÖR USA: USA
 BOXNR.: 1937

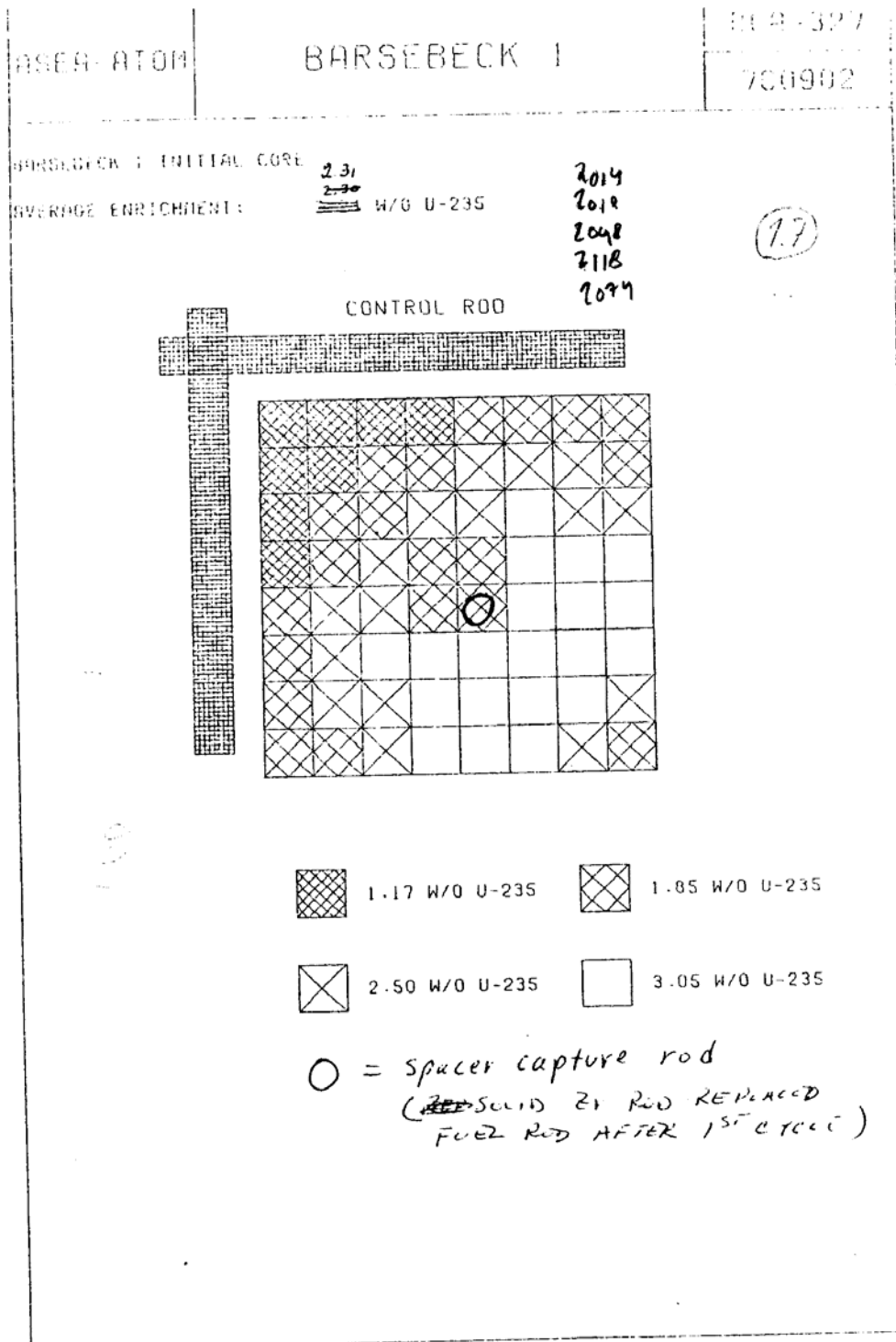
Initialhärden B1.

BIL. 1



Stavtyp nr	Typ	Anrikn. W % U235	Utbränningsregion
1	Smal	1,17	1
2	"	1,17	2
3	Normal	1,17	2
4	"	1,85	3
5	Smal	1,85	3
6	Normal	1,85	4
7	"	2,50	5
8	"	2,50	5
9	Smal	2,50	5
10	Normal	3,05	6
11	"	3,05	7

Barsebäck 1 Enrichment diagram assemblies 2014, 2018, 2048, 2074 and 2118



Barsebäck 1 Fuel rod diagram assemblies 9329

M O N T E R I N G S P R O T O K O L L 9329

058 * 9187 *

TOPPL: 24190 BOTTLPL: 24904 BOX: TRPSKY REF: HORN

*	1.98	2.49	2.49	2.49	2.49	1.98	1.98	1.38
*	2.936	2.974	3.288	3.280	3.286	3.285	2.986	2.991
8*	5561	5170	21892	25186	25190	20529	5533	6353
*	*****							
*	2.49	3.50	3.50	3.50	3.17BA	2.49	2.49	1.98
*	2.976	3.230	3.282	3.298	3.263	3.284	3.285	2.981
7*	5198	24332	24407	488	15427	21876	21887	5638
*	*****							
*	3.50	3.17BA	3.50	2.49	2.49	3.17	2.49	1.98
*	3.283	3.250	3.288	3.277	3.278	3.288	3.283	3.288
6*	24301	15496	24385	21812	21851	25044	21822	20534
*	*****							
*	3.50	3.50	3.50	1.38	1.38	2.49	3.17BA	2.49
*	3.285	3.297	3.286	3.280	3.287	3.279	3.262	3.284
5*	25184	458	24499	20162	20003	21982	15464	25178
*	*****							
*	3.50	3.50	3.50		1.38	2.49	3.50	2.49
*	3.287	3.290	3.284		3.295	3.281	3.294	3.283
4*	25188	24469	24446	58042	20201	21859	477	25182
*	*****							
*	3.50	3.50	3.50	3.50	3.50	3.50	3.50	2.49
*	3.278	3.281	3.279	3.280	3.282	3.279	3.289	3.280
3*	24304	24459	24337	24336	24338	24280	24395	21834
*	*****							
*	3.50	3.17BA	3.50	3.50	3.50	3.17BA	3.50	2.49
*	2.985	3.262	3.279	3.281	3.299	3.255	3.279	2.977
2*	6121	15574	24343	24322	451	15485	24350	5092
*	*****							
*	2.49	3.50	3.50	3.50	3.50	3.50	2.49	1.98
*	2.987	2.990	3.280	3.282	3.282	3.280	2.971	2.981
1*	5076	6076	24282	25180	25176	24348	5196	5520

	A	B	C	D	E	F	G	H

ANRIKN.%		ANTAL	KG	KG	KG	UFAKTOR
NOM	VERKL	STAVAR	U02	U	U-235	
1.38	1.395	4	12.853	11.328	0.159	0.8814
1.98	1.990	6	18.507	16.311	0.324	0.8814
2.49	2.496	18	57.553	50.728	1.262	0.8814
3.17BA	3.180	5	16.302	14.073	0.448	0.8633
3.17	3.172	1	3.288	2.898	0.092	0.8814
3.50	3.517	29	94.659	83.433	2.938	0.8814

S:A 2.922 65 203.162 178.771 5.223

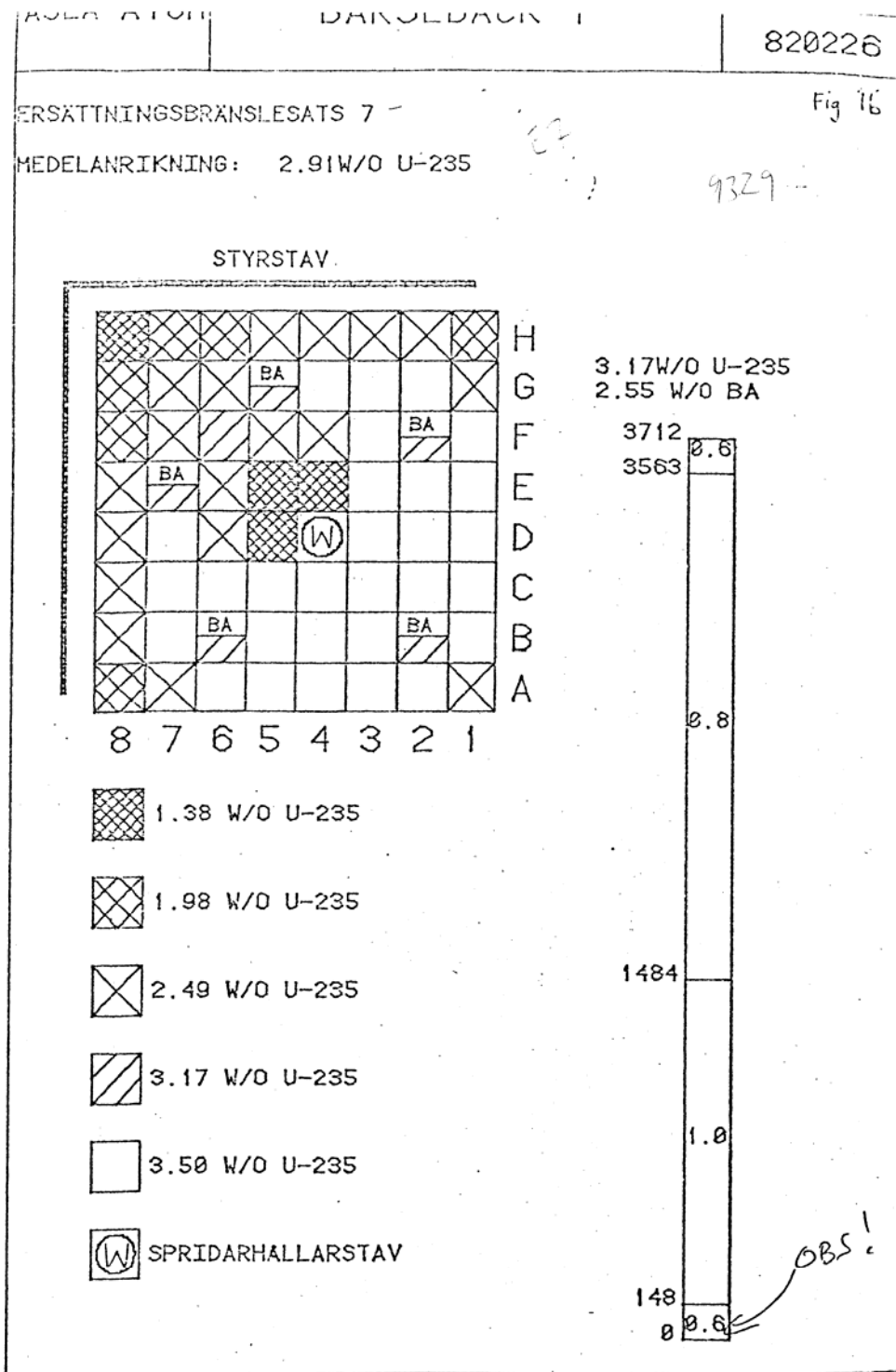
BA-HALT = 2.55 W/O I BA-KUTSAR

SPRIDARE NR.: 56820 56737 56748 56712 56734 56731

1983-02-16

GDDK. FOR INFRYSNING OCH LEVERANS DAT.....SIGN:.....

Barsebäck 1 Enrichment diagram assembly 9329



MONTERINGSPROTOKOLL BWR STANDARD
 MONTERAD DEN: 84-01-02 AV GE

									REF-HÖRN	
8	I	5174	6287	20689	20825	20718	24405	5205	5982	I
	I	02	04	05	05	05	03	02	01	I
7	I	6261	22785	22722	443	15289	20728	20775	5166	I
	I	04	06	06	07	08	05	05	02	I
6	I	22746	15313	22668	22647	22579	22700	20869	24467	I
	I	06	08	06	06	06	06	05	03	I
5	I	22690	412	22675	22626	22560	22642	15281	20807	I
	I	06	07	06	06	06	06	08	05	I
4	I	22666	22627	22659	71034	22667	22676	251	20813	I
	I	06	06	06	09	06	06	07	05	I
3	I	22678	22657	22614	22604	22695	22694	22681	20842	I
	I	06	06	06	06	06	06	06	05	I
2	I	6220	15272	22692	22664	387	15271	22774	6195	I
	I	04	08	06	06	07	08	06	04	I
1	I	6216	6191	22658	22663	22802	22843	6215	5087	I
	I	04	04	06	06	06	06	04	02	I

PATRON-NUMMER/-TYP : 10288 1
 PROJEKT-NUMMER/-NAMN: 071 B1E8

KAT	TYP	ANRIKNINGAR		
01	H	1,38		
02	H	1,98		
03	N	1,98		
04	H	2,49		
05	N	2,49		
06	N	3,32		
07	B	3,32		
08	BA	3,17	3,17	3,17
09	SP			

SPRIDARE 1(TOP): 63532
 SPRIDARE 2 : 63515
 SPRIDARE 3 : 63516
 SPRIDARE 4 : 63547
 SPRIDARE 5 : 63333
 SPRIDARE 6(BOT): 63503

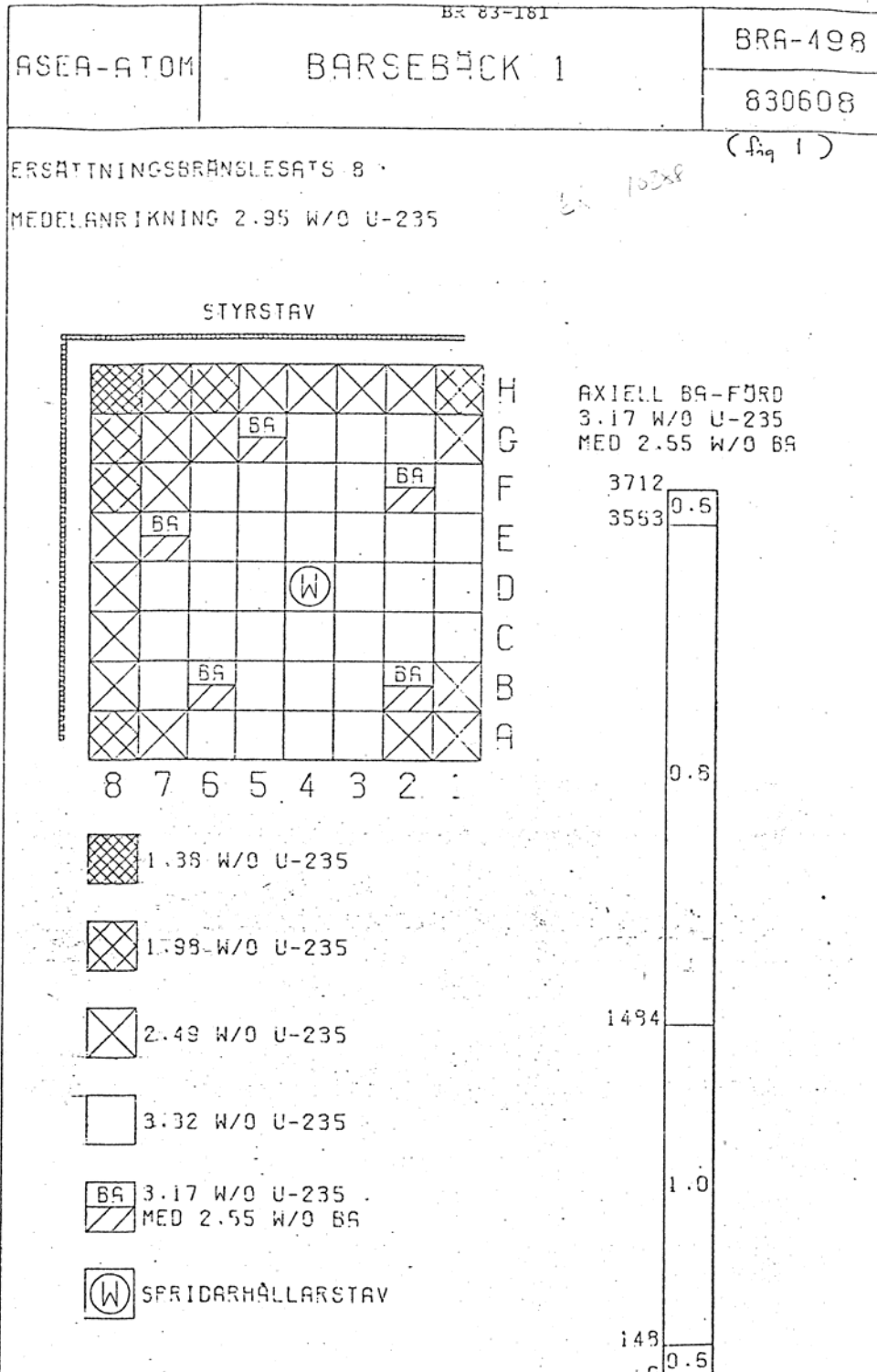
TOPPLATTA: 74382
 TJ.BLADFJ 1: 9198/3:2/578
 TJ.BLADFJ 2: 9198/1:2/572
 TUNN.BLADFJ: 9198/5:3/610
 M7-SKRUV LANG: 9306/2:3
 M7-SKRUV KORT: 9306/1:3

BOTTENPLATTA: M2394
 M8-MUTTER: M8-49
 Y.TR.FJÄDER: 9075:1/518
 BOX: 318012L

LEVERANS GODKÄND (ATOM/PCB) DATUM: 1984-02-01 SIGNATUR: *[Signature]*

10288

Barsebäck 1 Enrichment diagram assembly 10288



2.10 Barsebäck 2

REACTOR BARSEBÄCK 2

Parameter	Value
Reactor pressure (MPa) at full power	7
Inlet temperature into core (C) at full power	260
Outlet temperature into core (C) at full power	286
Pitch between assemblies in the core (c-c, mmm)	153
Avg. temperature in the uranium pellets at full power (C)	650
Avg. temperature in the cladding at full power (C)	290
Temperature in the boxwall at full power (C)	286

Cycle data

Cycle no	Start	Stop	Reactor power MW	Burnup avg/cycle MWd/tU	No of assemblies in core
9	1987-09-19	1988-07-06	1,800	6,050	444
10	1988-08-07	1989-09-08	1,800	8,090	444
11	1989-09-28	1990-07-11	1,800	5,630	444
12	1990-08-18	1991-09-06	1,800	8,110	444
13	1991-09-22	1992-07-02	1,800	5,600	444
14	1992-08-08	1993-09-30	1,800	5,350	444
15	1994-01-29	1994-06-29	1,800	2,820	444

Fuel data

Fuel data	Fuel id 14076
Fuel type	AA8x8
No of fuel rods	63
Rod pitch normal rods (mm)	16.3
Rod pitch normal – corner rods (mm)	16.05
Rod pitch corner – corner rods (mm)	15.8
Rod diameter normal rod (mm)	12.25
Clad thickness normal rod (mm)	0.8
Pellet diameter normal rod (mm)	10.44
No of normal rods	51
Rod diameter corner rod (mm)	11.75
Clad thickness corner rod (mm)	0.8
Pellet diameter corner rod (mm)	9.94
No of corner rods	12
Active length (mm)	3,712
Density UO ₂ (g/cc)	10.44
Porsity, dishing etc (%)	1
	10.3356
No of spacer rods	1
Material in spacer rods	Zr ₂
Outer diameter spacer rods (mm)	12.25
Cladding thickness spacer rod	
No of water rods	
Material in water rods	
Outer diameter water rods (mm)	
Inner diameter water rods (mm)	
No of BA rods	5
% Gd ₂ O ₃	2
Rel poison	0.8–1
No of spacers	6
Spacer material	Inconel
Mass of spacer (g)	0135
Box material	Zr ₄
Box outer measure square (mm)	139
Box wall thickness (mm)	2.3

Initial data		17076
Initial mass Utot (g)		179,571.00
Initial mass U235 (g)		5,665.00
Avg.enrichment% U235		3.15
Fuel rod diagram		X
Enrichment diagram		X
Data after rebuild 1		
Date of rebuild		
No of fuel rods		
No spacer rods		
No water rods		
No of water holes		
Mass Utot after rebuild (g)		
Mass U235 after rebuild (g)		
Data after rebuild 2		
Date of rebuild		
No of fuel rods		
No spacer rods		
No water rods		
No of water holes		
Mass Utot after rebuild (g)		
Mass U235 after rebuild (g)		
Cycle history burnup/cycle, MWd/tU		
4		
5		
6		
7		
8		
9		8,398
10		10,218
11		5,514
12		9,993
13		5,884
Axial BU distribution, EOL MWd/tUNode		
1		21,600
2		34,500
3		40,500
4		42,800
5		44,700
6		45,600
7		45,700
8		45,200
9		45,600
10		45,500
11		45,100
12		44,400
13		44,600
14		44,300
15		43,700
16		42,700
17		42,600
18		42,000
19		41,000
20		39,300
21		38,000
22		35,600
23		31,700
24		26,500
25		20,400

Void history		14076
Node no (top=25)		
1		0.00
2		0.00
3		0.00
4		0.05
5		0.13
6		0.20
7		0.27
8		0.33
9		0.39
10		0.44
11		0.49
12		0.53
13		0.56
14		0.59
15		0.62
16		0.64
17		0.66
18		0.68
19		0.70
20		0.72
21		0.73
22		0.74
23		0.75
24		0.76
25		0.76
Density history		
Node no (top=25)		
1		0.76
2		0.76
3		0.74
4		0.70
5		0.65
6		0.60
7		0.55
8		0.50
9		0.46
10		0.42
11		0.39
12		0.37
13		0.34
14		0.32
15		0.31
16		0.29
17		0.28
18		0.26
19		0.25
20		0.24
21		0.23
22		0.23
23		0.22
24		0.22
25		0.21

Control rod history
Node no (top=25)

1
2
3
4
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